

# Background Information Document To Support NESHAPS Rulemaking On Nuclear Power Reactors

40 CFR Part 61
National Emission Standards
For Hazardous Air Pollutants



40 CFR Part 61 National Emission Standards for Hazardous Air Pollutants

# BACKGROUND INFORMATION DOCUMENT TO SUPPORT NESHAPS RULEMAKING ON NUCLEAR POWER REACTORS

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U.S Environmental Protection Agency
Office of Radiation and Indoor Air
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## PREFACE

In the Federal Register notice dated August 5, 1991, the Administrator of the Environmental Protection Agency proposed, pursuant to the 1990 amendments to the Clean Air Act (CAA), to rescind the National Emission Standard for Hazardous Air Pollutants (NESHAP) promulgated on October 31, 1989, as it applies to commercial nuclear power reactors (40 CFR 61, Subpart I)

The 1990 amendments to the CAA allow the Administrator of EPA (the Administrator) to decide not to regulate NRC-licensed facilities under Section 112 of the CAA if the Administrator determines that NRC's existing programs for regulating these facilities assure that the public's health is being protected with an ample margin of safety

This Background Information Document (BID) was prepared to support the rescission rulemaking by presenting information concerning the NRC's regulatory program and the results of dose assessments performed for all operable commercial nuclear power reactors. Copies of the BID are available to all interested persons, an announcement of the availability appears in the *Federal Register*. For additional information, contact Eleanor Thorton at (202) 233-9773 or write to

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# Chapter 1

### INTRODUCTION AND EXECUTIVE SUMMARY

### 1.1 STATEMENT OF PURPOSE

In the Federal Register notice dated August 5, 1991, the Administrator of the Environmental Protection Agency (EPA) proposed, pursuant to the 1990 amendments to the Clean Air Act (CAA), to rescind the National Emission Standard for Hazardous Air Pollutants (NESHAP) promulgated on October 31, 1989, as it applies to commercial nuclear power reactors (40 CFR 61, Subpart I) The purpose of this Background Information Document (BID) is to present information relevant to the Administrator's reconsideration of the need for a NESHAP to control radionuclides emitted to the air from commercial nuclear power reactors.

The 1990 amendments to the CAA allow the Administrator of EPA (the Administrator) to decide that no regulation of NRC-licensed facilities is needed under Section 112 of the CAA if the Administrator determines that NRC's existing programs for regulating these facilities ensure that the public's health is being protected with an ample margin of safety.

The Administrator has determined that an ample margin of safety exists for commercial nuclear power reactors, therefore, the EPA is taking the final action of rescinding Subpart I of the NESHAP, as it applies to commercial nuclear power reactors

### 1.2 SCOPE

This BID presents information on the relevant portions of the regulatory framework that NRC has implemented for nuclear power plant licensees, under the authority of the Atomic Energy Act, as amended, to protect the public's health and safety. To provide context, it summarizes the rulemaking history for Subpart I. It then describes NRC's regulatory program for routine atmospheric emissions of radionuclides and evaluates the doses caused by actual airborne emissions from nuclear power plants, including releases resulting from anticipated operational occurrences

Specifically, the NRC programs in the following areas are summarized

- Airborne Emissions Controls the basis for establishing gaseous effluent limits,
- <u>Licensing</u> the criteria for establishing reactor design, construction, and operating limits for airborne emissions, and the procedures used to provide reasonable assurance that as-built reactors are consistent with the licensing basis provided by the design criteria,

- <u>Airborne Emissions Monitoring</u> the requirements for monitoring reactor effluents and environmental media to ensure that plant airborne emissions conform with the licensing basis;
- <u>Inspection</u> the combination of special (regional) and onsite inspections for ensuring that each reactor's licensing basis for allowable airborne effluents is being maintained; and,
- <u>Enforcement</u> the mechanisms available to ensure that licensees correct any deviations from design bases and operating limits

An inherent part of comparing plant performance with the licensing basis is the evaluation of the likelihood that NRC's policies and practices will continue to maintain or improve the plants' level of effectiveness. Therefore, additional information is presented on NRC's programs to detect and correct deficiencies (or make improvements) in plant desig loostruction, and operation.

The BID also evaluates airborne emissions from civilian power reactors plus those of the Federal power authorities. Doses to the most exposed individuals and the populations residing within 80 kilometers of each operable nuclear power station in the United States are estimates based on actual emissions data and the CAP-88 computer codes. The trend in airborne emissions over the last 15 years is also evaluated. Appendix D presents a list of all commercial reactors (and their status -- whether operating, under construction, or no longer in service) considered in this study.

# 13 ORGANIZATION OF THE BID

The remainder of this report is organized as follows. Chapter 2 summarizes the history of the NESHAPs rulemaking as it pertains to power reactors, Chapter 3 discusses NRC's programs intended to protect the public's health and safety, and Chapter 4 presents the methodology and results of the evaluations made of airborne emissions from all operable nuclear power reactors and the trend of airborne emissions over the last 15 years.

# 1.4 SUMMARY AND CONCLUSIONS

NRC's regulatory program has been reviewed, and those elements pertinent to the control of airborne emissions of radioactive material from operations have been described. Additionally, actual radioactive airborne emissions from over 100 power reactors at 69 reactor sites were obtained and the resulting doses to nearby individuals and the populations residing within 80 kilometers of the sites were evaluated using the CAP-88 computer codes

# The major findings of this study include

- The doses received by any member of the public from airborne emissions of radionuclides from operable reactors, as estimated using the CAP-88 computer codes, are less than 1 millirem per year effective dose equivalent (mrem/yr ede) The doses being received by the members of the public at greatest risk are lower than the NESHAP standard (10 mrem/yr ede with not more than 3 mrem/yr ede caused by radioiodines) established by the Administrator
- While the doses received by members of the public are expected to vary from year to year at any given reactor site, the trend for all reactors shows a decline in the collective population dose over the past 15 years
- NRC begins to consider doses received by the public from radioactive effluents early in the licensing process and continues to evaluate doses throughout a plant's lifetime. Each applicant must provide evaluations of the proposed reactor design and effluent controls against the ALARA design criteria in Appendix I of 10 CFR 50
- The results of the Appendix I evaluations are reflected in the allowable radioactive releases incorporated into each operating plant's Radiological Effluent Technical Specifications (RETS)
- Each plant is required to perform effluent monitoring, with results submitted to the NRC semiannually, to provide assurance that actual effluents are within the limits established by the RETS
- Each plant is required to calculate doses to the most exposed individuals residing near the plants on a semiannual basis to confirm that the ALARA objectives of Appendix I are being achieved
- 7. Each plant is required to monitor environmental media to confirm that the assumptions used in the calculation of offsite doses are appropriate for the site.
- NRC conducts an extensive inspection program, including onsite resident inspectors, specialized regional inspectors, and independent environmental monitoring, to confirm that the plants are operating in conformance with their regulations and limits

# Chapter 2

# SUMMARY OF THE RADIONUCLIDE NESHAP RULEMAKINGS FOR NRC-LICENSED AND URANIUM FUEL CYCLE FACILITIES

# 2.1 RULEMAKINGS UNDER THE CLEAN AIR ACT AMENDMENTS OF 1977

Pursuant to the 1977 amendments to the Clean Air Act (CAA), on December 29, 1979, the Administrator listed radionuclides as a hazardous air pollutant under Section 112 of the Act (44 FR 76738). The Administrator then initiated studies to determine what source categories of facilities emit radionuclides to the air in quantities sufficient to warrant establishing a NESHAP (National Emission Standard for Hazardous Air Pollutants) to limit emissions to levels providing an ample margin of safety to protect the public health

On April 6, 1983, EPA published a FR notice proposing radionuclide NESHAPs for four source categories and announced its finding that NESHAPs were not required for seven of the source categories that it had investigated (48 FR 15076) NESHAPs were proposed to limit emissions of radionuclides from elemental phosphorus plants, Department of Energy (DOE) facilities, certain non-fuel cycle facilities licensed by the NRC, and underground uranium mines. Uranium fuel cycle facilities were one of the seven source categories that the Administrator determined did not require a NESHAP.

In October 1984, acting pursuant to a court order to take final action on the proposed NESHAPs, the Administrator published a FR notice announcing that the proposed standards for elemental phosphorus plants, DOE facilities, and certain NRC-licensed facilities were being withdrawn (49 FR 43906). The decision to withdraw the proposed standards was based on the Administrator's finding that control practices already in effect for those source categories provide an ample margin of safety. The FR notice also made final the Administrator's decision not to issue NESHAPs for the other seven source categories

The decision to withdraw the proposed NESHAPs was immediately challenged in court, and on December 11, 1984, the U.S. District Court for the Northern District of California found the Administrator in contempt of its earlier order directing the Administrator to promulgate final standards or to make a finding that radionuclides are not a hazardous air pollutant. EPA complied with the court's December decision by issuing NESHAPs for elemental phosphorus plants, DOE facilities, and certain NRC-licensed facilities on February 6, 1985 (50 FR 7280)

The Environmental Defense Fund (EDF), the Natural Resources Defense Council (NRDC), and the Sierra Club filed petitions with the court to review the final decisions not to regulate certain source categories (including the uranium fuel cycle) and the February 1985 standards. On July 28, 1987, while these petitions were pending, the US Court of

Appeals for the District of Columbia remanded to the Agency the NESHAP for vinyl chlorides (a nonradioactive hazardous air pollutant). In that decision, the court concluded that the Agency had improperly considered cost and technological feasibility in determining the level of the standard without first making a determination based exclusively on the risk to the public

Given the court's decision on vinyl chloride, EPA determined that its radionuclide NESHAPs should also be reconsidered and petitioned the court for a voluntary remand of standards. In its petition, EPA also moved that the pending litigation on all issues relating to its radionuclide NESHAPs be placed in abeyance during the rulemaking and agreed to reexamine all issues raised by the parties to that litigation. The court granted EPA's petition on December 8, 1987, and EPA began to revisit its earlier decision under a courtimposed schedule.

Th. 'dministrator's final deciries on the radionuclide NESHAPS were published in the Federal Register on December 15, 1989 (54 FR 51654) The final NESHAP for the NRC-licensed facilities (40 CFR 61, Subpart I) included facilities that are part of the uranium fuel cycle and established a standard of 10 mrem/yr ede to any member of the public, with no more than 3 mrem/yr ede caused by emissions of radioiodines. In explaining his decision to promulgate a NESHAP that included the uranium fuel cycle facilities previously unregulated under the CAA, the Administrator explained that the standard would insure that the current levels of emissions do not increase

The December 15, 1989, FR notice also announced that the Administrator was granting a reconsideration of the NESHAP for NRC-licensed facilities (40 CFR 61, Subpart I). The reason for the reconsideration was to allow assessment of information received late in the rulemaking process from NRC and the National Institutes of Health (NIH) regarding the impacts of duplicative regulations on licensees and the potential for the NESHAP to discourage the use of radioisotopes in medical and experimental therapies. In granting the reconsideration, the Administrator also stayed the effective date of Subpart I for 3 months EPA subsequently extended the stay of effectiveness date of Subpart I on several occasions

# 2.2 RULEMAKINGS UNDER THE CLEAN AIR ACT AMENDMENTS OF 1990

While Subpart I was under reconsideration, the Congress passed the 1990 amendments to the CAA. The amendments include a new Section 112(d)(9) which states, in part

No standard for radionuclide emissions from any category or subcategory of facilities licensed by the Nuclear Regulatory Commission (or an Agreement State) is required to be promulgated under this section if the Administrator determines, by rule, and after consultation with the Nuclear Regulatory Commission, that the regulatory program established by the Nuclear Regulatory Commission pursuant to the Atomic Energy Act for such category or subcategory provides an ample margin of safety to protect the public health

In March 1991, EPA issued an Advance Notice of Proposed Rulemaking (ANPR), announcing its intention to proceed with the rulemaking, pursuant to Section 122(d)(9), to rescind Subpart I of 40 CFR 61, as it applies to nuclear power reactors (56 FR 10524). Concurrent with the ANPR, EPA published a FR notice proposing to stay the effectiveness of Subpart I for power reactors until the conclusion of the rulemaking on recision (56 FR 10523). On August 5, 1991, the proposed rule to rescind Subpart I as it applies to nuclear power reactors was published and public comments solicited (56 FR 37196).

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# Chapter 3

# THE NRC's REGULATORY PROGRAM

To determine whether the NRC's regulatory program is comprehensive, the following questions were asked

- Has the NRC established airborne effluent limits or design objectives consistent with the level of protection provided by the NESHAPs?
- Has the NRC's program for imposing design, construction, and operational
  performance limits on reactors resulted in reasonable assurance that as-built
  designs would be consistent with their design criteria and performance
  objectives?
- Does the NRC's program for monitoring reactor operations ensure that plant airborne emissions conform with the licensing basis?
- Does the NRC's program for periodic independent reviews in the operational period result in reasonable assurance that airborne emissions are kept as low as reasonably achievable?
- Does the NRC's enforcement policy provide incentive for the licensees to meet their regulatory obligations?

# 3.1 SETTING AIRBORNE EMISSION CONTROLS

Current regulations limiting routine radionuclide airborne emissions from commercial light water reactors are set forth in 10 CFR 20, Appendix I to 10 CFR 50, and 40 CFR 190 Part 20 establishes "Standards for Protection Against Radiation" The recent revisions to Part 20 establish a new limit of 100 mrem/yr ede for members of the public. The 100 mrem/yr limit covers doses from both gaseous and liquid effluents and considers exposures from all sources. Part 20 also imposes the requirement that exposures be as low as reasonably achievable (ALARA). Licensees may demonstrate compliance with this limit using the effluent concentrations set forth in Table 2 of Appendix B of 10 CFR Part 20 The values in Table 2 for air are based on 50 mrem/yr.

Appendix I to Part 50 establishes "Numerical Guides for Design Objectives and Limiting Conditions for Operation to Meet the Criterion As Low as is Reasonably Achievable for Radioactive Material in Light-Water-Cooled Nuclear Power Reactor Effluents" The Appendix I design objectives require an applicant for a permit to construct a nuclear power reactor to describe how radionuclide releases to unrestricted areas will be kept ALARA The applicant satisfies the design objectives, in part, by demonstrating that the

gaseous radionuclide releases to the atmosphere from each light water reactor on site will not result in an estimated average annual air dose in excess of 10 mrads for gamma exposure and 20 mrads for beta exposure. These limits, applicable to any unrestricted location that can be occupied, pertain only to external exposure to noble gases

Lower radionuclide release rates may be required to satisfy the design objectives if the releases are likely to result in an estimated annual external dose from airborne effluents in excess of 5 mrem/yr. Alternatively, higher release rates may be acceptable if the applicant can provide reasonable assurance that the external dose to any individual in unrestricted areas will not exceed 5 mrem/yr to the whole body and 15 mrem/yr to the skin. The applicant must also demonstrate that the calculated annual total quantity of all radioiodines and radioactive particulates to be released to the atmosphere from each light water reactor will not cause exposures to any individual in unrestricted areas in excess of 15 mrem to any organ.

The limiting conditions of operation (LCOs) set forth in Appendix I complement the design objectives. They provide guidance to ensure that, during operation, the facility maintains radionuclide releases and offsite exposures as low as reasonably achievable and consistent with the design objectives.

EPA's environmental radiation standards for nuclear power operations are set forth in 40 CFR Part 190 40 CFR Part 190 requires, in part, that the radiation doses to real individuals from all uranium fuel-cycle sources, including all gaseous and liquid effluent pathways and direct radiation, should not exceed 25 mrem/yr to the whole body or any organ, except the thyroid. The dose limit to the thyroid is established at 75 mrem/yr.

The guidelines set forth in Appendix I to 10 CFR Part 50 and the standards set forth in 40 CFR 190 together establish a regulatory framework assuring that the routine airborne radioactive emissions from commercial light water reactors will not result in exposures in excess of 10 mrem/yr ede. Specifically, the principal radionuclides routinely released in the airborne effluents from commercial light water reactors are noble gases and radioiodines. The whole-body dose from noble gas emissions per reactor is limited to 5 mrem/yr by 10 CFR 50, Appendix I. The organ doses from radioiodines and particulates are limited to 15 mrem/yr per reactor. For the thyroid gland, which is the critical organ for radioiodines, a dose of 15 mrem/yr yields an effective dose equivalent (ede) of less than 1 mrem/yr Therefore, the NRC's design objectives set forth in Appendix I limit the allowable dose to 6 mrem/yr ede, which is consistent with the NESHAP

# 3.2 LICENSING PROGRAM

Compliance with Appendix I and 40 CFR 190 is ensured by the Limiting Conditions of Operation (LCOs) and Radiological Effluent Technical Specifications (RETS) established in accordance with 10 CFR Part 50.36a. The LCOs and their associated RETS require that

the licensee take action if the quantity of radioactive materials actually released in airborne emissions to unrestricted areas in any calendar quarter is such that the resulting radiation exposure would exceed one-half the annual design objectives. The licensee is required to investigate the cause of the excess, define and initiate a program of corrective action, and report these actions to the NRC within 30 days from the end of the quarter during which the release occurred.

To assist licensees in complying with the LCOs and preparing their RETS, the NRC's staff has issued dozens of guidance documents. These documents provide highly detailed standard RETS and procedures for implementing the RETS. Guidance is also provided on ensuring compliance with 40 CFR 190

# 3.3 AIRBORNE EMISSIONS MONITORING

The LCOs and RETS require the licensee to initiate airborne emissions and environmental monitoring programs to provide data on (1) the quantities of radionuclides released, (2) the levels of radiation and radioactive materials in the environment, and (3) changes in land use and demography in the vicinity of the site that pertain to compliance with the LCOs. If the monitoring data reveal that the relationship between the quantities of radioactive materials released and the doses to individuals in unrestricted areas is significantly different than that assumed in the calculations used to assess compliance with the design objectives, the NRC's staff can require modifications of the RETS

Licensees are required to submit results of their monitoring programs to the NRC semiannually. In addition, the NRC conducts a program in which the direct radiation in the sectors surrounding each nuclear power facility is measured independently by NRC's dosimeters located in the same areas as the licensee's dosimeters.

The NRC also provides guidance documents on periodic reporting requirements pertaining to the LCOs

# 3.4 INSPECTION PROGRAM

In accordance with the NRC's "Inspection Manual," regional NRC inspectors inspect each facility about two times per year. In conjunction with the plant's required reports, the inspections determine the degree to which each plant is complying with its license and technical specifications, including its RETS. If problems are identified, followup inspections are scheduled in order to ensure that deficiencies are corrected. If a facility appears to have persistent problems in particular areas, inspections are performed more frequently

The periodic inspections of the RETS include a review of records and procedures, interviews with plant personnel, and an audit of airborne emissions and the environmental measurements program. The measurements program consists of the independent collection

of airborne emissions and environmental samples by NRC personnel and sample analysis using an NRC mobile laboratory

Each plant also has a full-time NRC Senior Resident Inspector who provides continual health and safety oversight of plant operations. If problems arise pertaining to compliance with the RETS, NRC's inspector may request a special inspection. Areas of noncompliance are punishable by both civil and criminal penalties

# 3.5 ENFORCEMENT PROGRAM

A license or a construction permit may be revoked, suspended, or modified, in whole or in part, for failure to follow any of the terms and provisions of the Atomic Energy Act (AEA), regulations, license, permit, or orders of the Commission

Addingonally, the NRC may ob ... an injunction or other court order prohibiting or violation of its rules, regulations, or orders, including the payment of civil penalties. Willful violations may be punished by fine or imprisonment or both, as provided by law

More information on the NRC and its programs is presented in Appendix A. Appendix B lists selected regulatory guides pertaining to power reactors. Guides applicable to power plant airborne emissions are highlighted.

# Chapter 4

# EVALUATION OF AIRBORNE EMISSIONS AND DOSES FROM POWER REACTORS

### 4.1 PURPOSE

Evidence of the effectiveness of the NRC's regulatory program was obtained by calculating the doses to the members of the public residing near operable nuclear power reactors. The trend in annual population doses is also presented to allow projections of likely future results.

Exposure estimates are presented for 69 nuclear reactor sites representing over 100 power reactors. Using site-specific data, exposures to the maximally exposed individuals and members of the public residing within a 50-mile (80-kilometer) radius were calculated for each of the 69 nuclear reactor sites

This chapter discusses the results of the evaluation and includes a description of the methodology used. The computer code data input sheets used in this analysis are provided in Appendix C. Key assumptions are discussed in the text.

# 4.2 METHODOLOGY

# 4.2.1 General Approach

Doses to nearby individuals and populations within 0 to 50 miles are estimated for all operable nuclear power plants using the CAP-88 methodology described in detail in Volume 1 of EPA 520/1-89-005 (EPA 89) In addition, trends in reactor performance are evaluated for the years since the enactment of Appendix I to 10 CFR 50.

Brookhaven National Laboratory, an NRC contractor, provided airborne emissions data for calendar year 1988 (NUREG/CR-2907, Volume 8) (NRC 89c) This was the most recent year for which a complete set of data was available when this report was prepared. The data, obtained by each plant from its effluent monitoring systems, reflect the total quantities of radionuclides released to the air from both anticipated occurrences and accidents. Examination of the data indicates that nearly all of the emissions are the result of planned releases from normal operation or from anticipated operational occurrences (NRC 89c). The latter include unplanned releases of radioactive materials from miscellaneous actions such as equipment failure, operator error, or procedure error. For normal operations, factors such as fuel cladding defects, design features of plant waste treatment systems, operational occurrences, and equipment performances strongly affect releases. Inspection of the data also indicates that only two plants (R.E. Ginna and Yankee Rowe) monitor and report releases of C-14. The remaining plants, while required to account for C-14 in determining compliance with their

Radiological Effluent Technical Specifications (RETS), are not required by the NRC to analyze for C-14 or include it in their environmental reports. The EPA has evaluated the significance of C-14 releases at commercial nuclear power plants and has determined: 1) C-14 release rates are reliably estimated based on power generation, and 2) C-14 does not contribute a significant amount to the maximum individual dose (SCA92).

The data set provided by Brookhaven were augmented as needed to provide a complete and representative set. Since not all power reactors operate in a given year, or they operate at low power levels or for relatively short periods of time, any given year's data for airborne emissions can be less than normal. To assure the data do not underestimate exposures, airborne emissions data from other years were obtained for all operable plants that had below normal airborne emissions in 1988. If 1989 or 1990 data were not available, prior years were researched until a complete year of data could be obtained. Appendix D identifies all plants analyzed and their operating status in 1990.

The airborne emissions data were combined with other site-specific data and were input to the CAP-88 computer codes used in this evaluation

# 4.2.2 Computer Codes

Radioactive releases from a nuclear power plant may contribute to radiation exposure through several external and internal exposure pathways. External exposures may result from direct cloud immersion or from radionuclides deposited on the ground. Internal exposure may result from inhalation of airborne radioactivity or from ingestion of contaminated food products. The magnitude of public exposure from a power reactor site is largely determined by the quantity of specific radionuclides contained in the airborne emissions and by a complex set of atmospheric dispersion and deposition processes

Computer codes are commonly used to model dispersion and deposition processes which determine human exposure. The EPA has developed a set of computer programs that estimate doses and health risk due to radionuclide emissions to the air. The computer software, referred to as the Clean Air Act Assessment Package - 1988 (abbreviated as CAP-88), estimates dose and health risks from the inhalation, ingestion, air immersion, and ground surface irradiation pathways and tabulates results for the maximally exposed individuals and populations at varying locations around a reactor site.

The computer code that models environmental dispersion and transport in CAP-88 is AIRDOS-EPA. This code uses a modified version of the standard Gaussian plume dispersion equation to compute radionuclide concentrations in air, rates of deposition on ground surfaces, concentrations in foods, and human intake rates resulting from inhalation and ingestion of contaminated air and food products. Modeling of internal dose resulting from ingestion of contaminated food products requires estimates of (1) radionuclide concentrations in food and (2) the quantity of foods consumed Estimates of the radionuclide concentrations in produce, milk, and meat are made by coupling the AIRDOS-EPA atmospheric transport

model data with input parameters defined by NRC Regulatory Guide 1.109 for the terrestrial food chain (NRC 77a)

# 4.2 3 Site-Specific Data

To estimate doses more accurately around each of the 69 power reactor sites, site-specific data were used, when possible, to replace average or computer-code default values. The following site-specific input parameters were determined for each of the 69 nuclear facilities:

- <u>Airborne Emissions Data</u> Annual airborne effluent releases, by radionuclide, were obtained from all operable nuclear reactors for the latest year (typically 1988) for which data were readily available
- Release Height The stack height (i.e., the airborne emissions release point) for each reactor was obtained Since data on discharge velocity and heat content could not be obtained readily, plume rise is not accounted for in the modeling
- <u>Meteorological Data</u> Meteorological data involving seasonal temperatures and precipitation were obtained from the National Climatic Data Center based on records for the 30-year period 1951 to 1980, inclusive. Joint frequency distributions, wind speed, and wind direction by stability class were obtained from the nearest airport
- Population Distribution Two separate sources of information were used to establish population distribution within each of 16 sectors within a 50-mile radius of the plant site. For distances greater than 5 miles, the SECPOP data base, which contains 1980 Census data, was used For radii of less than 5 miles, SECPOP was not considered sufficiently accurate due to the widely varying size of the census enumeration districts. Population distributions for distances of less than 5 miles were obtained from Final Safety Analysis Reports (FSARs) filed by utilities as part of the NRC's licensing process. Invariably, FSARs contain projected population distributions, typically in 10-year increments. In most instances, 1990 population data within 5 miles of the power reactor site were used to supplement the SECPOP data.
- Nearest Resident Because the nearest residents are often the maximally exposed, and because their locations are subject to change, current and accurate data were sought. Therefore, for each site, the most recent semiannual Radiological Environmental Monitoring Program (REMP) report was consulted to find the distance to the nearest resident in each of the 16 sectors.
- Regional Food Consumption The computer code (PREPAR) was used to input data to CAP-88 regarding local consumption of food products PREPAR integrates data concerning regional agricultural productivity, population distribution, meteorological data, and information concerning the fraction of food assumed to be

home-grown, produced within the assessed 50-mile radius of a facility, and imported from outside the 50-mile assessed area.

# 4.3 SUMMARY OF RESULTS

# 4.3.1 Recent Airborne Emissions and Doses

The doses to the population within a 50-mile radius and the effective dose equivalents to the maximally exposed individuals (MEI) that were estimated for each of 69 licensed nuclear reactor sites are presented in alphabetical order for pressurized water reactors (PWRs) and boiling water reactors (BWR, in Tables 4-1 and 4-2, respectively. These tables also present the effective dose equivalents attributable to individuals exposed to radioiodines and the number of persons within the 50-mile radii.

Average population doses were calculated by dividing the collective effective dose equivalent by the total number of individuals residing within the area bounded by a 50-mile radius from the plant site.

Tables 4-1 and 4-2 also provide a power reactor capacity factor extracted from the Licensed Operating Reactors - Status Summary Report (NRC 89d). This number is obtained by dividing the net electrical energy generated in 1988 by the designed electrical rating, and then multiplying by the total number of hours in the year The capacity factor values provide an approximate means of estimating doses for conditions of maximum power generation.

- 4.3.1.1 Summary. Estimated doses cited in Tables 4-1 and 4-2 are analyzed and summarized in Tables 4-3 and 4-4 which provide the range, median, and distribution of the effective dose equivalents for PWRs and BWRs. The following conclusions are drawn from the estimated doses:
  - 1. The highest dose to the maximally exposed individual at any of the facilities is estimated to be approximately 1 mrem/yr, or 10 percent of the 10 mrem/yr NESHAP limit. The average dose to an individual residing within a 50-mile radius of a nuclear power facility was less than 0 0012 mrem/yr
  - 2. Estimated doses for PWR facilities are comparable to those for BWR facilities
  - 3. Radioiodines contributed a very small fraction to the effective dose equivalent of the maximally exposed individuals Exposures to radioiodine are well below the NESHAP limit of 3 mrem/yr ede.

Table 4-1. CAP-88 results for 1988<sup>(1)</sup> airborne emissions from pressurized water reactors (PWR)

AVEDACE	STATION	MEI	MEI			
AVERAGE	CAPACITY	(ALL NUCLIDES)	(IODINES)	COLLECTIVE DOSE	POPULATION .	IND. DOSE
REACTOR SITE	FACTOR (%)	(mrem/year)	(mrem/year)	(person-rem/year)	W/IN 50 MILES	(mrem/year)
REACTOR STILE	FACTOR (A)	(IIII CIIV Yea) /	(All City year /	(person remy year y	W/111 30 111220	Cim diay / dai /
Alvin W. Vogtle 1 & 2 <sup>(2)</sup>	71.6	3.E-3	5.E-7	5.E-2	526,623	1.E-4
Arkansas Nuclear One 1 & 2	59.7	9.E-3	1.E-5	6.E-2	200,820	3.E-4
Beaver Valley 1 & 2	79.3	7.E-4	3.E-6	6.E-2	3,562,306	1.E-5
Brandwood 1 & 2	74.3	4.E-4(L)	1.E-5	3.E-2	4,813,692	6-E-6(L)
8vron 1 & 2	64.8	2.E-3	1.E-4	8.E-2	935,515	9.E-5
Calloway	90.8	4.E-3	6.E-7	3.E-2	350,743	1.E-4
Caivert Cliffs 1 & 2	81.2	3.E-2	6.E-3(H)	1.E+0	2,513,028	4.E-4
Catawba 1 & 2	69.6	6.E-3	1.E-5	1.E-1	1,419,084	1.E-4
Crystal River 3	80.0	7.E-4	1.E-6	2.E-2	384,023	6.E-5
Davis-Besse	15.4(L)	6.E-4	4.E-6	9.E-2	1,789,403	5.E-5
Diablo Canyon 1 & 2	60.6	9.E-4	5.E-6	4.E-2	269,576	1.E-4
Donald C. Cook 1 & 2	54.2	2.E-3	1.E-4	4.E-2	1,110,161	3.E-5
Fort Calhoun	62.6	2.E-3	1.E-5	4.E-2	742,481	6.E-5
H. B. Robinson 2	54.4	6.E-3	1.E-4	5.E-2	670,611	8.E-5
Haddam Neck	62.6	2.E-2	1.E-3	5.E-1	3,385,414	1.E-4
Indian Point 2 & 3	79.9	1.E-1(H)	1.E-4	2.E+0	15,215,971(H)	1.E-4
Joseph M. Farley 1 & 2	91.0	1.E-2	2.E-5	1.E-1	336,357	4.E-4
Kewaunee	88.6	5.E-4	4.E-6	1.E-2	601,485	2.E-5
Maine Yankee	70.5	4.E-4	1.E-5	1.E-2(L)	579,421	2.E-5
McQuire 1 & 2	77.9	7 E-3	8.E-5	2.E-1	1,583,564	1.E-4
Milistone 1, 2 & 3 <sup>(4)</sup>	82.9	3.E-3	1.E-4	2.E-1	2,525,597	9.E-5
North Anna 1 & 2	92.5	1.E-2	2.E-4	3.E-1	1,037,163	3.E-4
Oconee 1, 2 & 3	83.9	5.E-2	8.E-4	9.E-1	918,009	1.E-3
Pal 1sades	53.6	4.E-3	2.E-4	8.E-2	1,080,138	8.E-5
Palo Verde 1, 2 & 3	41.6	2.E-2	4.E-4	1.E+0	886,595	1.E-3
Point Beach 1 & 2	88 4	3.E-3	1.E-5	1.E-1	596,702	1 E-4
Prairie Island 1 & 2	87.5	1.E-3	3.E-8	4.E-1	2,125,846	2.E-4
R. E. Ginna	85.6	3.E-3	6.E-6	3.E-1	1,182,809	2.E-4
Rancho Seco	36.7	6.E-3	4.E-6	3.E-1	1,811,338	1.E-4
Salem 1 & 2	68.9	3.E-3	3.E-6	2.E+O(H)	4,693,004	5.E-4
San Onofre 1, 2 & 3	65.6	1.E-2	3.E-4	1.E+0	4,730,036	4.E-4
Sequoyah 1 & 2 <sup>(3)</sup>	73.5	4.E-2	3.E-5	1.E+0	810,824	1.E-3(H)
Shearon Harris	70.5	1.E-2	1.E-9(L)	2.E-1	1,279,315	2.E-5
South Texas Project 1 & 2 <sup>(2)</sup>	72.1	6.E-2	6.E-5	1.E-1	193,540(L)	7.E-4
St Lucie 1 & 2	92.7(H)	4.E-2	8.E-4	7.E-1	536,404	1.E-3
Surry 1 & 2	45.6	6.E-3	3.E-4	3.E-1	1,697,490	2.E-4
Three Mile Island 1	79.4	7.E-3	3.E-5	3.E-1	2,056,038	1 E-4
Trojan	65.9	1.E-3	4.E-5	1.E-1	1,353,571	9.E-5
Turkey Point 3 & 4	57.4	5.E-3	4.E-5	1.E+0	2,146,801	5.E-4
Virgil C. Summer	65.0	3.E-3	1.E-4	2.E-2	814,083	3.E-5
Waterford 3	69.2	3.E-3 2 E-2	8.E-5	1.E+0	1,809,591	1.E-3
Wolf Creek	67.2	1.E-2	6.E-6	1.E+0 1 E-1	498,606	3.E-4
Yankee	76.1	3.E-3	8.E-7	8.E-2	1,512,433	5.E-5
21on 1 & 2	63.4	3.E-3 4.E-3	6.E-6	0.E-2 1.E+0	6,963,054	2.E-4
TIMI I B E	03.4	4.6-3	0.5-0	1.570	0,703,034	Z.E 7

<sup>1988</sup> emissions data were used in calculating offsite doses for all power plants except as designated by footnotes (2) and (3).

<sup>1990</sup> emissions data used in calculating offsite doses.

<sup>(3) 1983</sup> emissions data used in calculating offsite doses.

<sup>(4)</sup> Millstone Unit 1 is a BWR; however, the emissions data were combined with that for Units 2 and 3 which are PWRs.

<sup>(</sup>L) indicates lowest value within the column.

<sup>(</sup>H) indicates highest value within the column.

Table 4-2. CAP-88 results for 1988<sup>(1)</sup> airborne emissions from boiling water reactors (BWR)

NUCLEAR POWER PLANT	STATION CAPACITY FACTOR (%)	ME1 (ALL NUCLIDES) (mrem/year)	MEI (IODINES) (mrem/year)	COLLECTIVE DOSE (person-rem/year)	POPULATION W/IN 50 MILES	AVERAGE IND. DOSE (mrem/year)
Big Rock Point	63.9	1.E-1	4.E-5	1.E-1	156,984(L)	7.E-4
Browns Ferry 1, 2 & 3(2)	49.8	9.E-1(H)	4.E-4	1.E+1(H)	691,349	2.E-2(H)
Brunswick 1 & 2	60.4	9.E-3	1.E-4	9.E-2	221,157	4.E-4
Clinton	71.7	2.E-3	5.E-6	3.E-2	871,185	4.E-5
Cooper	62.6	1.6-2	3.E-4	1.E-2	176,268	8.E-5
Dresden 2 & 3	62.5	8.E-3	6.E-4	7.E-1	6,377,923	1.E-4
Duane Arnold	68.8	4.E-3	3.E-5	1.E-1	580,018	2.E-4
Edwin I. Hatch 1 & 2	62.5	9.E-3	5.E-5	1.E-1	292,839	3.E-4
Fermi 2	45.0	8.E-4	1.E-5	6.E-2	5,058,443	1.E-5
Grand Gulf	95.6	4.E-3	1 2-6(L)	2.E-3(L)	251,084	1.E-5
Hope Creek	74.8	1.E-3	0.E+0	5.E-1	5,356,435	9.E-5
•						
James A. Fitzpatrick	63.5	1.E-2	4.E-5	2.E-1	841,917	2.E-4
LaSalle County 1 & 2	61.0	2.E-2	4.E-4	3.E-1	1,068,100	3.E-4
Limerick 1	72.0	3.E-3	2.E-4	6.E-1	6,849,011(H)	9.E-5
Monticello	97.1	2.E-2	3.E-4	3.E-1	2,111,208	1.E-4
Nine Mile Point 1 & 2 <sup>(3)</sup>	33.3(L)	1.E-3	1.E-5	5.E-2	841,367	6 E-5
Oyster Creek	65.0	4.E-2	4.E-4	1.E+0	3,428,989	5.E-4
Peach Bottom 2 & 3 <sup>(3)</sup>	77.8	2.E-4	7.E-6	7.E-2	4,342,379	1.E-5
Perry 1	68.4	1.E-2	3.E-3(H)	1.E-1	2,401,506	7.E-5
Pilgrim <sup>(4)</sup>	91.5	2.E-2	2.E-4	9.E-1	4,231,393	2.E-4
Quad Cities 1 & 2	72.8	6.E-3	2.E-4	9.E-2	697,505	1.E-4
River Bend	88.2	1.E-4(L)	1.E-5	5.E-3	723,760	7.E-6(L)
Susquehanna 1 & 2	78.9	5.E-4	1.E-5	4.E-2	1,534,873	3.E-5
Vermont Yankee	92.9(H)	4.E-3	3.E-5	1.E-1	1,310,563	9.E-5
Washington Nuclear Project 2	62.4	5.E-3	4.E-4	2.E-1	249,129	8.E-4

<sup>(1)</sup> 1988 emissions data were used in calculating offsite doses for all power plants except as designated by footnotes (2), (3), and (4)

<sup>1983</sup> emissions data used in calculating offsite doses. (3)

<sup>1990</sup> emissions data used in calculating offsite doses.

<sup>1985</sup> emissions data used in calculating offsite doses.

<sup>(</sup>L) indicates lowest value within the column.

<sup>(</sup>H) indicates highest value within the column.

4.3.1.2 Uncertainties In making these dose estimates, EPA has tried to give best estimates of radionuclide concentrations in the environment and in specific tissues in arriving at effective dose equivalent values EPA's CAP-88 computer programs are well established and have been shown to agree well with experimental data. In addition to airborne emissions data reported by individual facilities, site-specific data including stack height, meteorology, population distribution, and agricultural productivity were used. Nevertheless, uncertainties exist regarding all dose estimates. The level of uncertainty can be assumed to be greatest in the estimate of the maximum individual at risk. For population dose estimates, variabilities in the parameter producing the uncertainties tend to cancel out.

The average estimate of dose when <u>all</u> facilities are considered is not likely to differ significantly from year to year Comparisons of average annual station capacity factors for the past 16 years (Table 4-5) show that the overall capacity factor for the entire nuclear power plant industry is relatively consistent

# 4.3.2 Trends for Airborne Emissions and Doses

Although population doses have not been documented for all previous years, collective population doses for every facility have been estimated for the years 1975 through 1987 (NRC 90c). Because plant émissions directly affect offsite doses, annual trends in emissions and population doses can serve as indicators of dose to the average individual as well as to the maximally exposed individual. Therefore, reductions in emissions and/or collective population doses imply a reduction in individual doses, including the MEI

Figures 4-1 through 4-4 depict annual airborne emission trends from 1975 through 1986 associated with (1) fission and activation gases and (2) iodines and particulates for PWRs and BWRs. Annual collective population dose trends are presented in figures 4-5 and 4-6, respectively, for PWR and BWR sites for the years 1975 through 1987 Since at many of these sites, station capacity factors have increased and the total population within 50 miles has increased in proportion to national average values (about 1 percent per year), the associated reductions in population exposures suggest a parallel and perhaps even stronger trend in individual exposures

The generic trend in emissions and collective dose reductions since 1975 is likely the result of improved fuel performance and plant operations. For BWRs, the reduction in the amount of airborne effluents released since the mid-1970s is due largely to the installation of augmented off-gas (AOG) systems, many of which were required to meet the provisions of Appendix I to 10 CFR 50, promulgated by NRC in May 1975

Table 4-3 Statistical summary of effective dose equivalents for PWR nuclear generating stations

Range			ge
Effective Dose Equivalents	Low	High	Overall Average ± 1 SD
MEI - all nuclides (mrem)	0.0004	0 103	0.013 ± 0.021
MEI - iodines (mrem)	1.21E-9	0.006	0 0003 ± 0.0010
Population Dose (person-rem)	0 0124	2 440	0.470 ± 0.657
Average lucividual Dose (mrem)	0 00002	0 001	0 0003 ± 0 0004

Table 4-4 Statistical summary of effective dose equivalents for BWR nuclear generating stations

	Range		
Effective Dose Equivalents	Low	High	Overall Average ± 1 SD
MEI - all nuclides (mrem)	0 0001	0 989	$0.053 \pm 0.0196$
MEI - iodines (mrem)	0.0	0.004	0.0003 ± 0.0007
Population Dose (person-rem)	0.0026	1 840	0.241 ± 0.393
Average Individual Dose (mrem)	0.000003	0 0009	0 0002 ± 0 0002

Table 4-5 Annual average station capacity factors\*

<u>Year</u>	Capacity Factor (%)
1973	60.3
1974	55.7
1975	64 1
1976	62 3
1977	68.2
1978	70 4
1979	63 4
1980	61 4
1981	62 4
1982	60 0
1983	59.2
1984	60.2
1985	62 9
1986	60 1
1987	63 7
1988	65.1

<sup>\*</sup> Data for 1973 - 1987 were taken from reference NRC 90d Data for 1988 were taken from reference NRC 89d

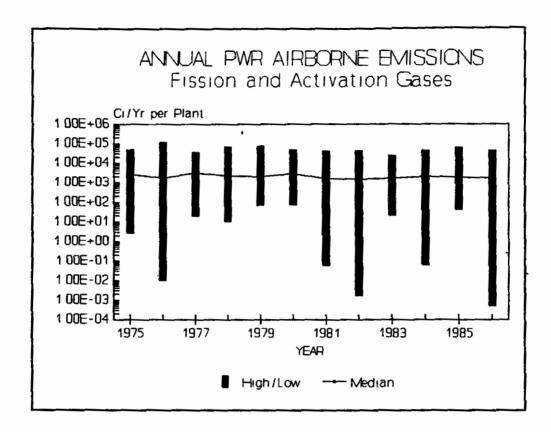


Figure 4-1

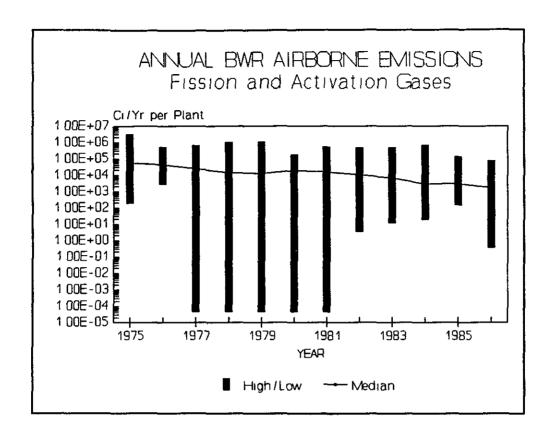


Figure 4-2

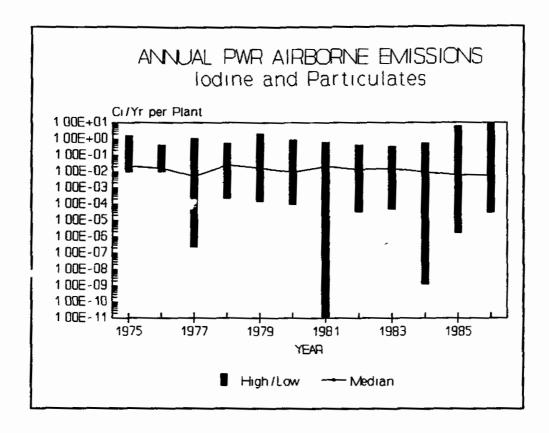


Figure 4-3

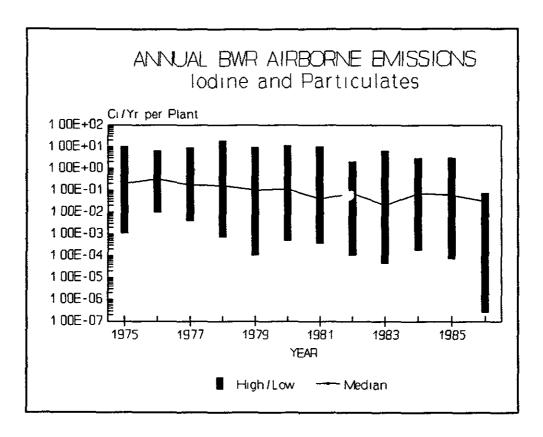


Figure 4-4

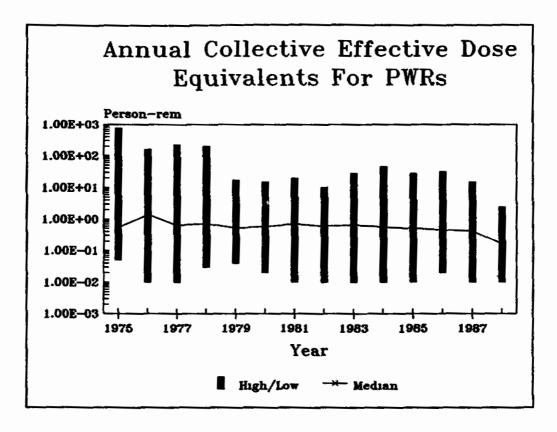


Figure 4-5

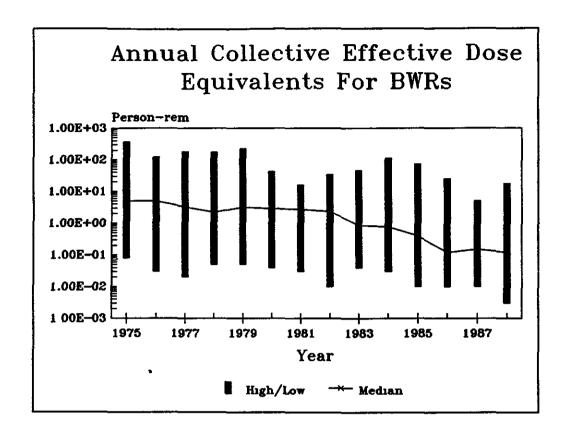


Figure 4-6

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#### REFERENCES

- ANS 75 American Nuclear Society 18 1 Working Group "Radioactive Material in Principal Fluid Streams of Light-Water-Cooled Nuclear Power Plants" American National Standards Source Term Specification N237 1975
- EEI 86 Edison Electric Institute "A Report on the Management Structure of the Nuclear Regulatory Commission Prepared for the Edison Electric Institute"

  June 1986
- EPA 88 U S Environmental Protection Agency. "Limiting Values of Radionuclide Intake and Air Concentrations and Dose Conversion Factors for Inhalation, Submersion, and Ingestion" (also known as "Federal Guidance Report No 11") EPA-520/1-88-020 September 1988
- EPA 89 U S Environmental Protection Agency "Risk Assessments Methodology, Environmental Impact Statement, NESHAPs for Radionuclides Background Information Document " EPA/520/1-89-005, Volume 1. September 1989
- ICRP 77 International Commission on Radiological Protection "Recommendations of the International Commission on Radiological Protection" ICRP Publication 26, Annals of the ICRP Vol 1, No 3 Pergamon Press, New York 1977
- KEM 79 Report of the President's Commission on the Accident at Three Mile Island
  "The Need for Change The Legacy of TMI" 1979
- NCRP 87 National Council on Radiological Protection "Public Radiation Exposure From Nuclear Power Generation in the United States" NCRP-92 December 1987.
- NRC 72 U.S Nuclear Regulatory Commission "Onsite Meteorological Programs" Regulatory Guide 1 23. March 1972
- NRC 73

  U.S Nuclear Regulatory Commission "Final Environmental Statement Concerning Proposed Rule Making Action Numerical Guides for Design Objectives and Limiting Conditions of Operation to Meet the Criterion 'As Low As Practicable' for Radioactive Material in Light Water Cooled Nuclear Power Reactor Effluents "WASH-1258 July 1973
- NRC 74

  U.S Nuclear Regulatory Commission "Measuring, Evaluating, and Reporting Radioactivity in Solid Waste and Releases of Radioactive Material in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants" Regulatory Guide 1 21, Revision 1. June 1974

- NRC 76a U.S Nuclear Regulatory Commission. "Calculation of Releases of Radioactive Materials' in Gaseous and Liquid Effluents from Light-Water-Cooled Power Reactors." Regulatory Guide 1 112 April 1976.
- NRC 76b U.S. Nuclear Regulatory Commission "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Pressurized Water Reactors (PWR-GALE Code) " NUREG-0017 April 1976
- NRC 76c U.S. Nuclear Regulatory Commission "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Boiling Water Reactors (BWR-GALE Code)." NUREG-0016. April 1976.
- NRC 76d U.S. Nuclear Regulatory Commission. "Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors." Regulatory Guide 1.110. March 1976.
- NRC 77a U.S Nuclear Regulatory Commission. "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR 50, Appendix I." Regulatory Guide 1.109, Revision 1. October 1977.
- NRC 77b U.S. Nuclear Regulatory Commission. "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Power Reactors." Regulatory Guide 1.111, Revision 1. July 1977.
- NRC 77c
  U.S. Nuclear Regulatory Commission. "XOQDOQ, Program for the Meteorological Evaluation of Routine Effluent Releases at Nuclear Power Stations" Report 0324 September 1977.
- NRC 78 U.S. Nuclear Regulatory Commission "Preparation of Radiological Effluent Technical Specifications for Nuclear Power Plants" NUREG-0133 October 1978.
- NRC 83a U S Nuclear Regulatory Commission "Standard Radiological Effluent Technical Specifications for PWRs" NUREG-0472 January 1983
- NRC 83b U.S. Nuclear Regulatory Commission "Standard Radiological Effluent Technical Specifications for BWRs." NUREG-0473. January 1983
- NRC 87 U.S. Nuclear Regulatory Commission. "Radioactive Materials Released from Nuclear Power Plants" NUREG/CR-2907 1987.

- NRC 89a U.S Nuclear Regulatory Commission "The 1989 NRC Annual Report" NUREG-1145, Vol 6 July 1989
- NRC 89b U.S Nuclear Regulatory Commission. "John B Martin, NRC Regional Administrator, Region V, to Sacramento Municipal Utility District, Docket No. 50-312" EA 86-110 January 8, 1989
- NRC 89c U S Nuclear Regulatory Commission "Radioactive Material Released from Nuclear Power Plants" NUREG/CR-2907, Vol 8 1989.
- NRC 89d U S Nuclear Regulatory Commission. "Licensed Operations Reactors Status Summary Report, Data as of 12-31-88." NUREG-0020, Vol 13, No. 1 1989
- NRC 90a U S Nuclear Regulatory Commission "Nuclear Regulatory Commission Fact Sheet." NUREG/BR-0099, Revision 4 January 1990
- NRC 90b U S Nuclear Regulatory Commission. "U S Nuclear Regulatory Commission Functional Organization Charts " NUREG-0325, Revision 14 August 15, 1990
- NRC 90c U S Nuclear Regulatory Commission. "Population Dose Commitments Due to Radioactive Releases From Nuclear Power Plant Sites in 1987" NUREG/-CR-2850, Vol 9 1990
- NRC 90d U S Nuclear Regulatory Commission "Occupational Radiation Exposure at Commercial Nuclear Power Reactors and Other Facilities 1987." NUREG-0713, Vol. 9 1990
- PNL 87 Battelle Pacific Northwest Laboratories. "Fuel Performance Annual Report for 1985" NUREG/CR-3950, Vol 3. February 1987
- PNL 88 Battelle Pacific Northwest Laboratories. "Fuel Performance Annual Report for 1986" NUREG/CR-3950, Vol 4 1988.
- ROG 80 U.S Nuclear Regulatory Commission Special Inquiry Group. "Three Mile Island A Report to the Commissioners and to the Public." 1980.
- SCA 92 Memorandum to Al Colli (U.S EPA) from John Mauro (SC&A) regarding NESHAPS and Carbon-14, September 16, 1992.

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#### APPENDIX A

## NRC's ORGANIZATION, REGULATIONS, AND CONTROLS

#### A 1 ORGANIZATION AND RESPONSIBILITIES OF NRC

The NRC regulates the civilian uses of source, byproduct, and special nuclear materials in the United States. This mission is accomplished through the development and implementation of controls (i.e., rules, regulations, guidance, etc.) governing licensed activities; licensing of nuclear facilities (i.e., issuance of permits and licenses) and the possession, use, and disposal of nuclear materials; and inspection and enforcement activities to ensure compliance with these controls and the conditions imposed through permits and licenses

## A.1.1 Basic Functions

The NRC's responsibilities include protecting public health and safety, protecting the environment; protecting and safeguarding materials and plants in the interest of national security; and ensuring conformity with antitrust laws. During fiscal year 1990, the NRC had approximately 3,200 employees and a budget of over \$400 million to carry out three basic functions regulatory research and standards development, licensing, and inspection and enforcement

As part of its regulatory research and standards development function, the NRC is mandated by law to conduct an extensive confirmatory research program in the areas of safety, safeguards, and environmental assessment. The Commission establishes regulations, standards, and guidelines governing the various licensed uses of nuclear facilities and materials

In its licensing function, the agency reviews and issues licenses for the construction and operation of nuclear power plants and other nuclear facilities, and it licenses the possession and use of nuclear materials for medical, industrial, educational, research, and other purposes Regulatory authority for certain nuclear materials licensing has been transferred to certain states under the Agreement States Program authorized by the AEA. However, NRC retains authority for licensing and regulating nuclear power plants

NRC's inspection and enforcement activities include various kinds of inspections and investigations designed to ensure that licensed activities are conducted in compliance with its regulations and other requirements. NRC enforces compliance as necessary.

# A.1.2 Organization

A.1.2.1 The Commission. The Commission is composed of five members, appointed by the President and confirmed by the Senate, one of whom the President designates as Chairman. The Chairman is the principal executive officer of, and the official spokesman for the NRC, as mandated by the Reorganization Plan No 1 of 1980 (NRC 90b). The Advisory Committee on Reactor Safeguards (ACRS), which was assigned a statutory role by Congress, independently reviews and reports on safety studies and applications for construction permits and operating licenses. The ACRS advises the Commission with regard to hazards at proposed or existing reactor facilities and the adequacy of proposed reactor safety studies. On its own initiative, the ACRS may review specific generic matters or nuclear facility safety issues.

A.1.2.2 NRC Offices. The NRC reorganized in 1987 to reflect progressively less involvement with the construction of large, complex nuclear facilities and increased involvement with the operation and maintenance of these facilities

Office of Nuclear Reactor Regulation (NRR). The primary responsibilities of this office are to conduct the inspection and licensing activities associated with operating power reactors, including contractors and suppliers for such facilities. The Office also is responsible for evaluating applications to build and operate new power reactors, for inspection and licensing activities related to the construction and operation of research and test reactors, and for licensing reactor operators. In addition, the Office is responsible for inspecting NRC-licensed activities under its jurisdiction to ensure that they comply with all NRC regulations and requirements. NRR identifies and takes action regarding conditions and licensee performance that may adversely affect public health and safety or the environment, and assesses and recommends or takes action regarding incidents and accidents. Project Directorates within NRR monitor daily operations for all power reactors

Important elements of this program, as it applies to the construction and operation of nuclear power plants, include the Resident Inspection and Systematic Assessment of Licensee Performance (SALP) programs.

With respect to power plant air emissions, this Office is responsible for reviewing design data for effluent control and monitoring equipment and operating procedures.

Office of Nuclear Material Safety and Safeguards (NMSS) This Office has no responsibilities for power plant air emissions

Office of Nuclear Regulatory Research (RES) This Office has three primary responsibilities: (1) to plan, recommend, and implement programs of nuclear regulatory research, standards development, and resolution of safety issues of nuclear power plants and other facilities regulated by NRC; (2) to develop and promulgate all technical regulations; and (3) to coordinate research activities within and outside the agency including appointment of staff to committees and conferences

With respect to power plant air emissions, this Office is responsible for the promulgation and revision of regulations affecting emissions, such as 10 CFR Part 50, Appendix I, and 10 CFR Part 20 Additionally, the Office manages the development of regulatory guides

Office for Analysis and Evaluation of Operational Data (AEOD) This Office independently analyzes and evaluates operational safety data associated with NRC-licensed activities to identify issues that require action at the NRC or industry. Its other responsibilities include the reactor performance indicators program and the management and direction of programs for diagnosing evaluations and investigations of significant operational events

This Office evaluates semiannual plant airborne emissions data and unusual events that contribute to airborne emissions

Office of Enforcement This Office develops policies and programs for enforcement of NRC's requirements. It manages major enforcement actions and assesses the effectiveness and uniformity of enforcement actions taken by the Regional Offices. Enforcement powers include notices of violation, fines, and orders for license modification, suspension, or revocation.

Regional Offices. The NRC's five regional offices execute the established NRC policies and assigned programs relating to inspection, enforcement, licensing, state agreements, state liaison, and emergency response within each region. Each regional Division of Reactor

Projects inspects and evaluates assigned NRC programs for power reactors and implements the NRC's Resident Inspector Program and related enforcement actions for assigned facilities. The Division of Radiation Safety and Safeguards performs inspections and evaluations in radiological safety and environmental monitoring

With respect to airborne emissions, the Office of Nuclear Reactor Regulation administers and conducts the national NRC Thermoluminescent Dosimeter Program, in which the direct radiation in the sectors surrounding each nuclear power facility is measured independently by NRC's dosimeters located in the same areas as the licensee's dosimeters. The results of measurements for each reactor site from this NRC monitoring network are published quarterly in NRC documents available to the public

### A.2 CONTROLS APPLICABLE TO LICENSEES - GENERAL

## A.2.1 Establishing Airborne Emission Controls

This section describes the NRC's procedures for setting nuclear power plant controls to protect the health and safety of the public. These controls may take several forms rules and regulations; regulatory guides, generic letters, bulletins, and information notices; and NRC reports. The first two categories of controls for nuclear power plants are administered by the Office of Nuclear Regulatory Research (RES); the others are administered by the Office of Nuclear Regulation (NRR).

A.2.1.1 Rulemaking and Regulatory Guides. The term rulemaking actually covers the establishment of two kinds of regulatory documents - the regulations of NRC contained in Title 10 of the Code of Federal Regulations (10 CFR) and regulatory guides. The decision to move forward with either a rule or a regulatory guide is based upon the results of a regulatory analysis [itself based upon a Technical Findings Document (e.g., NUREG)]. Thereafter, both types of documents, rules and guides, are developed in a process that provides for internal and external (public) review and comment. The entire process is repeated again for the final rule or guide developed in light of comments received from the public.

Both types of documents are prepared in a two-step process. In the first step, a draft is produced for public comment. RES usually develops such drafts in consultation with and on behalf of NRR, NMSS, or both. The drafts are developed at a technical staff level,

coordinated up through parallel management chains of the affected offices, reviewed by the appropriate advisory committee (usually the ACRS except for waste management matters which now have their own advisory committee), reviewed by a senior management review group called the Committee for the Review of Generic Requirements (CRGR), and then presented to the appropriate decisionmaker(s) for action

When the development of a rule or a guide reaches the point where it is presented to the decisionmakers, the process diverges. Substantive rules can be issued for public comment only by a majority vote of the five NRC Commissioners. Therefore, proposed rulemakings are recommended for action by RES, with the concurrence of the affected program office, through the NRC's Executive Director for Operations, to the Commission. The Commission requests input from the appropriate advisory committees and the CRGR to assist in its decision.

Once the Commission has decided to issue a proposed rule for public comment, a notice of the proposed action is issued in the *Federal Register*, the notice also identifies the time allowed for comments and may specify particular questions on which the Commission desires input. These particular questions often involve the matters treated in the regulatory analysis performed for the proposed rule; e.g., the anticipated costs and other impacts of imposing the new rule.

The RES staff, in consultation with the affected program office, evaluates public comments received on a proposed rule. The Commission has used both rulemaking hearings, which are formal adjudicatory proceedings, and public meetings, which are less formal, to further discussion and obtain additional information concerning a proposed rule. Once the additional information has been received and evaluated, the staff modifies the rule as necessary, repeats the entire review process followed for the proposed rule, and returns the rulemaking package to the Commission for final action. When the Commission makes its final decision on the rule, it is issued as "effective" with a notice in the Federal Register. The rule then becomes a part of Title 10 of the Code of Federal Regulations

The process followed by the RES in developing a draft and then a final regulatory guide is essentially the same as that for a rule, except that the Executive Director for Operations and the Commission are not involved Rather, the final decision authority for

issuing regulatory guides, either in draft form for public comment or in final form, is the Director of the Office of RES

A.2.1.2 Generic Letters, Bulletins and Information Notices Generic letters, bulletins, and information notices are written NRC notifications sent to groups of licensees that identify specific problems, developments, or other matters of interest to the licensees. In some cases the NRC is calling for or recommending that the licensees take specific steps

A.2.1.3 NRC Reports NRC reports (usually referred to generically as NUREGs) are prepared by the NRC's staff, contractors, or national laboratories and provide the technical basis for decisionmaking. Special categories of such reports include Safety Evaluation Reports (SERs), Environmental Impact Statements (EISs), and Standard Review Plans (SRPs) The NRC issues the first two categories of reports to establish the conditions under which the license to construct or operate will be issued. The SRPs are issued to disseminate information about the regulatory licensing process and to improve the general public's and the nuclear industry's understanding of the staff's review process.

Standard Review Plans address the responsibilities of the persons performing the review, the matters that are reviewed, the Commission's regulations and acceptance criteria necessary for the review, how the review is accomplished, the conclusions that are appropriate, and the implementation requirements

## A.2.2 Licensing Program

Licensing programs utilize a system of controls, compliance guidance, and independent review to establish (with reasonable assurance) the ability of a facility to meet performance requirements. Of particular relevance is the NRC's ability to establish and maintain an acceptable level of performance through (1) independent review to verify that regulatory criteria were correctly translated into design, construction, and operations documents and (2) monitoring of operating data.

The licensing process begins with the filing of a license application, consisting of general information, an Environmental Report, and a Safety Analysis Report (SAR). The general content requirements of the SAR are contained in 10 CFR 50.34.

The NRC initiates a comprehensive technical review of the license application and any supporting documents after initial acceptance review and docketing. During this period, the NRC's staff and the Advisory Committee on Reactor Safeguards (ACRS) conduct independent technical reviews of the license application, resulting in the issuance of a Safety Evaluation Report (SER) by the NRC's staff and a formal letter of recommendation from the ACRS to the Chairman of the NRC

In determining whether to grant a construction permit, the NRC holds an adjudicatory public proceeding conducted by the Atomic Safety and Licensing Board (ASLB). At the end of the adjudicatory proceeding, the ASLB renders a decision supported by a written opinion. A decision of the ASLB could be appealed to an Atomic Safety and Licensing Appeal Board (ASLAB). The Commissioners, may also consider the matter upon a petition requesting such review. After all avenues of administrative appeal have been exhausted and if the ASLB's initial decision prevails, the Director of Nuclear Reactor Regulation issues a letter authorizing construction to begin

At least 18 months prior to anticipated completion of construction, the applicant submits an updated license application to the NRC in support of obtaining a license to operate. The NRC's staff and the ACRS again conduct technical reviews which, if favorable, result in the issuance of a Safety Evaluation Report by the NRC's staff and a formal letter of recommendation from the ACRS to the Chairman of the NRC

### A.2.3 Programs for Control During Design and Construction

As noted earlier, a primary responsibility of the NRC's Office of Nuclear Reactor Regulation (NRR) is to evaluate applications to build new power reactors. The following regulations and guidance have been developed to control that process during the design and construction phase. Portions pertaining to anticipated airborne emissions are highlighted

- A.2 3.1 Technical Contents of Application (10 CFR 50 34). The NRC requires each applicant to submit Preliminary and Final Safety Analysis Reports (PSAR/FSAR) to provide the most current, design-related, technical information for independent review. The NRC's review emphasizes the following areas:
  - (1) Safety assessment of the site including those site features affecting facility design, particularly the site evaluation factors identified in 10 CFR 100, "Reactor Site

Criteria." For airborne emissions, such factors include (a) population density and distribution, (b) meteorological conditions at the site and surrounding area (e.g., wind speed, direction, and dispersion), and (c) the use characteristics of the site environs (e.g., local food chain);

- (2) Description of the design and operating characteristics of the facility;
- (3) Facility design, including the principal design criteria, the design bases and the relation of the design bases to the principal design criteria, and information about materials of construction, general arrangement, and approximate dimensions, sufficient to provide reasonable assume need that the design will conform to the design bases with adequate margin of safety;
- (4) 10 CFR Part 50, Appendix A, "General Design Criteria"
  - Principal design criteria establish the necessary (but minimum) design, fabrication, construction, testing, and performance requirements for components to ensure safe operation. All departures from the General Design Criteria (GDC) must be identified and justified.
  - The GDC are divided into six categories, the last of which, "Fuel and Radioactivity Control," provides criteria for effluent controls. Specifically, Criterion 60, "Control of Releases of Radioactive Materials to the Environment," and Criterion 64, "Monitoring Radioactivity Releases," apply to airborne effluents.
    - Criterion 60 requires that the "...design shall include means to control ... the release of radioactive materials in gaseous .. effluents . . during normal reactor operation including anticipated operational occurrences"
    - Criterion 64 requires that "Means shall be provided for monitoring . . effluent discharge paths, and the plant environs for radioactivity that may be released from normal operations, including anticipated operational occurrences...."
    - Additionally, Criterion 1 (Quality Standards and Records) in GDC Category I (Overall Requirements) requires that (a) a quality assurance program be established and (b) that appropriate records be maintained throughout the life of the unit;
- (5) Analysis and evaluation of the design and performance of the components of the facility including a determination of margins of safety during normal operations and transient conditions anticipated during the life of the facility;

- (6) Identification of the kinds and quantities of radioactive materials expected to be produced in the operation and the means for controlling and limiting radioactive effluents. NRC's regulations limiting airborne radionuclide emissions from commercial light-water reactors are set forth in 10 CFR 20 and Appendix I to 10 CFR 50
  - 10 CFR Part 20, "Standards for Protection Against Radiation," specifies the release limits for radioactive material to unrestricted areas.
  - 10 CFR 34a, "Design Objectives for Equipment to Control Releases of Radioactive Material in Effluents - Nuclear Power Reactors"
    - Section 50.34a requires licensees to describe equipment to be installed and procedures to maintain control over radioactive materials in gaseous effluents produced during normal reactor operations, including expected operational occurrences. In addition, licensees must identify the design objectives and the means for keeping levels of radioactive material in effluents as low as reasonably achievable (ALARA). Numerical guidance for achieving ALARA releases is specified in 10 CFR Part 50, Appendix I, "Numerical Guides for Design Objectives and Limiting Conditions for Operation to Meet the Criterion 'As Low As Is Reasonably Achievable' for Radioactive Material in Light-Water-Cooled Nuclear Power Reactor Effluents."
    - Section 50 34a also requires licensees to estimate the quantity of each of the principal radionuclides of the gases, halides, and particulates expected to be released annually;
- (7) An identification and justification for the selection of those variables, conditions, or other items which are subjects of technical specifications,
- (8) The technical qualifications of the applicant, including personnel qualification requirements, and a plan for the training of personnel,
- (9) Plans for the conduct of normal operations, including maintenance, surveillance, and periodic testing;
- (10) Managerial and administrative controls to ensure safe operation;
- (11) A description of the quality assurance program
  - 10 CFR Part 50, Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants" (refer to Appendix E for a complete list and synopsis of QA program criteria)

- Quality assurance (QA) comprises all those planned and systematic actions necessary to ensure that a component will perform satisfactorily in service. This program must be documented by written policies, procedures, or instructions and must be carried out throughout the plant life. The QA program provides control over activities affecting the quality of components to an extent consistent with their importance to safety. The program must provide for the indoctrination and training of personnel performing activities affecting quality.
- Criterion XII, "Control of Measuring and Test Equipment," requires that measurements that affect the quality of work related to the accomplishment of the performance objectives and the design bases are taken only with instruments, tools, gauges, or other measuring devices that are accurate, controlled, calibrated, and adjusted at predetermined intervals to maintain accuracy within necessary limits
- Criterion XVII, "Quality Assurance Records," requires that "Sufficient records be maintained to furnish evidence of activities affecting quality. The records must include at least the following. Operating logs and the results of reviews, inspections, tests, audits, monitoring of work performance, and materials analysis. The records must also include closely related data such as qualifications of personnel, procedures, and equipment. Inspection and test records must, as a minimum, identify the inspector or data recorder, the type of observation, the results, the acceptability, and the action taken in connection with any deficiencies noted. Records must be identifiable and retrievable. Consistent with applicable regulatory requirements, the applicant must establish requirements concerning record retention, such as duration, location, and assigned responsibility";
- (12) The plan for coping with emergencies, including unplanned air releases.
- A.2.3.2 Conditions of Construction Permits (10 CFR 50.55) During the construction period, the licensee is subject to various terms and conditions to ensure that construction activities are conducted in accordance with the design bases and performance objectives agreed to in the PSAR. The NRC uses both active (NRC-initiated) and passive (licensee-initiated) regulatory requirements to maintain control during the construction phases.

Paragraph 50.55(e) requires the holder of the construction permit to notify the NRC of each deficiency found in design and construction, which, were it to remain uncorrected, could adversely affected the safety of operations.

A 2.3 3 Hearings and the ACRS (10 CFR 50.58) Each application for a construction permit (or operating license) is referred to the Advisory Committee on Reactor Safeguards (ACRS) for an independent review and report. The ACRS is a statutory body of scientists and engineers who advise the Commission on nuclear safety matters.

In addition to the views of the ACRS, the NRC seeks the views of the public and other interested parties by holding hearings on each application for a construction permit Public hearings on each application for an operating license may also be held

# A.2.4 Programs for Control During Operation

During the period of operation, the licensee is subject to various terms and conditions to ensure that activities are conducted in accordance with the design bases and performance objectives agreed to in the FSAR. As in the construction phase, the NRC uses both NRC-initiated and licensee-initiated regulatory requirements to maintain control during operations.

- A.2.4 1 Effluent Technical Specifications (10 CFR 50.36a). To keep releases of radioactive materials to unrestricted areas during normal reactor operations, including expected operational occurrences, as low as reasonably achievable, paragraph (a)(2) requires licensees to submit a semiannual report specifying the quantity of each of the principal radionuclides released to unrestricted areas in gaseous effluents during the previous 6-month period. This allows the Commission to estimate maximum potential annual radiation doses to the public resulting from releases. If releases are significantly above design objectives, the Commission may require the licensee to take action as the Commission deems appropriate.
- A.2.4 2 Changes, Tests, and Experiments (10 CFR 50.59). Once a license to operate has been issued, the NRC allows changes in facility design, operational procedures, and activities unless the proposed change involves a modification to the technical specifications or an unreviewed safety question. The licensee is required to maintain records and to report all changes in facility descriptions or procedures contained in the FSAR.
- A.2.4.3 Inspections (10 CFR 50 70) Each licensee (and holder of a construction permit) must permit NRC to inspect its records, premises, and activities. The licensee is required to provide office space onsite for a full-time NRC resident inspector. The resident inspector is afforded immediate unfettered access, equivalent to that allowed regular plant employees

A.2 4 4 Records and Reports (10 CFR 50 71) Each licensee and each holder of a construction permit is required to maintain records and make reports in accordance with the conditions established in the license or permit, or by the rules, regulations, and orders of the Commission. Record of power plant airborne emissions are maintained for the life of the facility.

In addition, each licensee is required to submit an annual update of its Final Safety Analysis Report (FSAR) to bring current (within 6 months) the material upon which the license was based.

- A 2.4.5 Notification Requirements (10 CFR 50.72). The NRC requires its licensees to notify it (sometimes) immediately) in the even. In addition to emergencie, there are notification requirements for non-emergency events. The following events require telephone notification followed by written reports:
  - (1) Any event or condition that alone could have prevented the fulfillment of the safety function of a component that controls the release of radioactive material;
  - (2) Any airborne radioactive release that exceeds 2 times the applicable concentration limits specified by regulation in unrestricted areas when averaged over a time period of 1 hour; and
  - (3) Any event or situation, related to the health and safety of the public (or onsite personnel), or protection of the environment.
- A.2 4.6 Licensee Event Report System (10 CFR 50.73). The holder of an operating license for a nuclear power plant is required by regulation to submit a Licensee Event Report (LER) within 30 days of the discovery of certain events. Events that may pertain to airborne emissions include:
  - (1) Any operation or condition prohibited by the plant's technical specifications;
  - (2) Any event that results in a condition outside the design basis of the plant; and
  - (3) Those ever 's described in the preceding section dealing with notification requirements

A.2.4 7 Backfitting (10 CFR 50.109) The Commission may require backfitting of a facility if it finds that such action is necessary to protect public health and safety or that it will provide substantial, additional protection at a justifiable cost

A.2 4.8 Other Programs. The Systematic Assessment of Licensee Performance (SALP) program is a principal and regular source of data used for judging licensee performance Under the SALP program, the performance of each nuclear power reactor licensee is evaluated through the periodic, comprehensive examination of all available data for each facility, including airborne emissions. The SALP review is intended to direct the NRC's and the licensee's management resources toward those areas that can most affect safety and that need improvement.

The SALP assessment includes a review of the past year's licensee event reports, inspection reports, enforcement history, and licensing safety issues. Also taken into account are evaluations by the NRC's resident and regionally based inspectors, licensing project managers, and senior managers. The SALP program supplements the normal regulatory processes. It is intended to provide meaningful guidance to utility management regarding the NRC's concerns about quality and safety in plant operation and construction (NRC 89a)

## A.2.5 Inspection Programs

Inspection programs use a system of unannounced and pre-planned visits to licensee and vendor facilities to permit personal observations of compliance with the licensing basis In addition, for power plant licensees, a resident inspector program has been established

Inspection is a basic element in the NRC's program. Inspections of licensed facilities determine the state of reactor safety, check that operations comply with the provisions of the license, and ascertain whether any unsafe conditions exist that would warrant corrective action. Both the NRC's headquarters and regional offices participate in inspections of operating reactors. The regional offices conduct most of the required inspections, using both regionally based and resident inspectors. In general, the regionally based inspectors are specialists, while resident inspectors are generalists. The Office of Nuclear Reactor Regulation is responsible for developing inspection policies and procedures and for monitoring and assessing the effectiveness and uniformity of the programs carried out by the headquarters and regional offices.

In addition to the routine or planned program of inspections, the NRC conducts a program (called "reactive" inspections) to deal with unsafe or potentially unsafe events or conditions at individual plant sites. In these inspections, the NRC seeks to determine the root cause of the event or condition, evaluates the licensee management's response to it, including action to prevent recurrence, and decides whether the problem is one that could occur at other facilities. The NRC then takes appropriate action.

A.2.5.1 Inspection of Licensees In accordance with the NRC's "Inspection Manual," each facility receives approximately two inspections per year that focus on reactor effluents. Specialist inspectors from the regional offices conduct these inspections. Considered with the plants' reporting requirements, the inspections determine the extent to which each plant is complying with its license and technical specifications, including its RETS. If problems are identified, followup inspections are scheduled to ensure that deficiencies are corrected. If a facility appears to have persistent problems in particular areas, inspections are performed more frequently and the resident staff may be increased.

The periodic inspections of the RETS include a review of records and procedures, interviews with plant personnel, and an effluent and environmental measurements program. The measurements program consists of the independent collection of effluent and environmental samples by NRC personnel from devices located offsite and usually adjacent to the licensees' instrumentation. Samples can be analyzed relatively quickly using an NRC mobile laboratory.

Each plant has a full-time NRC senior resident inspector who provides continual health and safety oversight of plant operations. Multiple reactor facilities may have more than one resident inspector. The resident inspector acts as the primary onsite evaluator for NRC's inspection efforts related to licensee incidents. If a problem arises within a particular area, the regional administrator can order a reactive inspection to investigate the area in detail (NRC 89a). If problems arise pertaining to compliance with the RETS, the NRC's resident inspector may request a special inspection or increase the frequency of inspections.

Areas of noncompliance are punishable by both civil and criminal penalties.

A.2.5.2 Inspection of Contractor and Vendor Programs. Much of the overall power plant design and construction effort is delegated to contractors and vendors <sup>1</sup> To ensure proper

<sup>&</sup>lt;sup>1</sup> "Contractor" is a term usually reserved for an entity that supplies services (e.g., design or construction) "Vendor" is a term usually reserved for an entity that supplies materials or finished products (e.g., pumps, valves, or concrete)

inspection of such activities, the NRC requires licensees to establish an element of control over the products supplied to it by contractors and vendors

The Vendor Inspection Program, which is centralized in NRC's headquarters, is principally a reactive program structured to respond to vendor and licensee reports of deviations or defects in vendor-supplied products, equipment, materials, and services NRC conducted 90 such vendor inspections in fiscal year 1989. The inspections dealt with a range of vendor activities involving plant operation, maintenance, procurements, and modifications Inspections of licensees, vendors, and contractors were triggered by information from a number of sources, such as licensee construction deficiency and operating reactor event reports, vendor reports of product defects, reports of events from the regional offices, allegations from members of the public concerning vendor activities, and vendor issues identified by NRC in its inspection programs

Control is exerted primarily through the NRC's requirements in 10 CFR 50, Appendix B, "Quality Assurance Criteria for Nuclear Power Plant and Fuel Reprocessing Plants."

## A.2.6 Enforcement Programs

The objective of the NRC's enforcement programs is to protect the public health and safety by ensuring that licensees comply with regulatory requirements. The NRC's enforcement policy, contained in 10 CFR Part 2, Appendix C, calls for strong enforcement measures to ensure full compliance and is designed to prohibit operations by any licensees who fail to achieve adequate levels of protection.

NRC's enforcement action has several levels of severity. The level of severity used in a given situation varies with the seriousness of the matter and the licensee's previous compliance record. The levels include

- Written Notices of Violation -- used in all instances of noncompliance with the NRC's requirements
- Civil penalties -- considered for licensees who evidence significant or repetitive instances of noncompliance, especially if a previous Notice of Violation has not been effective in achieving the expected corrective action. Civil penalties may also be imposed in the case of a particularly significant first-of-a-kind violation

• Orders to "cease and desist" operations, or for modification, suspension, or revocation of licenses — used in situations where licensees have not responded to civil penalties or where violations pose a significant threat to public health and safety or the common defense and security.

In fiscal year 1989, 75 enforcement cases resulted in civil penalty actions against power reactor licensees (NRC 89a). One of these cases involved the imposition of a civil penalty of \$100,000 on a power reactor licensee. The violations included failure to implement technical specification requirements to preclude release of liquid effluents containing radioactivity in amounts exceeding the dose criteria set forth in 10 CFR Part 50, Appendix I, and failure to maintain and implement procedures and report the release of radioactive material in effluents as prescribed in the facility Technical Specifications (NRC 89b). These violations were revealed during an inspection of the facility.

The following regulatory requirements are specific to power reactors:

- Revocation, Suspension, Modification of Licenses and Construction Permits for Cause (10 CFR 50 100) A license or a construction permit may be revoked, suspended, or modified, in whole or in part, for any material false statement in the application for license; or the revelation of new, adverse information; or for failure to construct or operate a facility in accordance with the terms of the construction permit or license; or for violation of, or failure to observe, any of the terms and provisions of the Act, regulations, license, permit, or orders of the Commission
- <u>Violations (10 CFR 50.110)</u> The NRC may obtain an injunction or other court order prohibiting any violation of its rules, regulations, or orders. Court orders may include the payment of civil penalties Additionally, any person who willfully violates any of the Commission's provisions may be guilty of a crime and, upon conviction, may be punished by fine or imprisonment or both, as provided by law.

### A.3 CONTROLS APPLICABLE TO AIRBORNE EMISSIONS

The NRC regulations limiting airborne radionuclide emissions from commercial light-water reactors are set forth in 10 CFR 20 and Appendix I to 10 CFR 50. Part 20 establishes "Standards for Protection Against Radiation," which are health-based standards and apply to all NRC-licensed facilities, not just reactors. Appendix I to Part 50 establishes "Numerical Guides for Design Objectives and Limiting Conditions for Operation to Meet the Criterion As Low as is Reasonably Achievable for Radioactive Material in Light-Water-Cooled Nuclear

Power Reactor Effluents " These are ALARA standards that reflect explicit consideration of both cost and benefits 2

## A.3.1 10 CFR 20

The portions of Part 20 that apply to airborne radionuclide emissions from licensed facilities are Part 20.1105, which sets permissible levels of radiation (in mrem per unit time) in unrestricted areas, and Part 20.1106, which establishes limits (in curies released to the environment) on radioactivity in effluents to unrestricted areas. Part 20.1105 states that the Commission will approve an application for a license to possess or use radioactive materials and any other source of radiation if the applicant can demonstrate that radionuclide releases are not likely to cause any individual in an unrestricted area to receive a dose to the whole body in excess of 100 mrem/yr. Part 20 1105 also requires that no individual in an unrestricted area receive a dose in excess of 2 mrem in any 1 hour or 100 mrem in any 7 consecutive days

Part 20.1106 limits, in part, the release of radioactive material to unrestricted areas to levels that will not result in average annual radionuclide concentrations in air and water in excess of the limits set forth in Table 2 of Appendix B of Part 20. This is a secondary standard designed to provide a level of assurance that the primary health-based standard of 100 mrem/yr ede is not exceeded.

In 1981, Part 20 was amended to adopt EPA standards set forth in 40 CFR Part 190 Part 190 requires, in part, that the radiation doses to real individuals from all uranium fuel cycle sources, including all gaseous and liquid effluent pathways and direct radiation, should not exceed 25 mrem/yr to the whole body or any organ, except the thyroid, which is limited to 75 mrem/yr

In addition to these numerical standards, Part 20 requires that each licensee make every reasonable effort to maintain radiation exposures, and releases of radioactive material in effluents to unrestricted areas, as low as is reasonably achievable. The term "as low as is

It should be noted that, since its promulgation in 1975, Appendix I to 10 CFR Part 50 establishes the most restrictive limits on the airborne effluents from light-water reactors. Theoretically, under some conditions, such as multiple reactor sites, the limits set forth in 40 CFR Part 190 for the nuclear fuel cycle could be more restrictive. However, in practice, Appendix I has been the limiting regulation. Appendix I and 40 CFR 190 are compared later in this section.

reasonably achievable," as defined in the Glossary of 10 CFR Part 20, means "as low as is reasonably achievable taking into account the state of technology, and the economics of improvement in relation to benefits to the public health and safety, and other societal and socioeconomic considerations in relation to the utilization of atomic energy in the public interest." Thus, the explicit consideration of cost is intended.

On January 9, 1986, major revisions to Part 20 (51 FR 1092) were proposed to keep pace with changes in the scientific knowledge underlying radiation protection that have occurred since Part 20 was originally issued more than 30 years ago. The revised rule implements the 1987 Presidential Guidance on occupational radiation protection and the recommendations of scientific organizations to establish risk-based limits and a system of dose limitation in accordance with the guidance published by the International Committee on Radiation Protection. At the time this report was prepared, the final revisions to Part 20 had just been promulgated, so it was necessary to base this summary on a pre-publication version of the rule, current as of March 1991.

Revised Part 20 requires that (1) the total effective dose equivalent to individual members of the public shall not exceed 100 mrem/yr, and 2 mrem in any 1 hour for external exposures, and (2) a licensee or applicant may apply for prior NRC authorization to operate up to an annual limit for an individual member of the public of 500 mrem. Part 20 also requires appropriate surveys to ensure that the dose limits are not exceeded. In addition, Table 2 of the revised Part 20 presents Derived Air Concentrations (DACs) that licensees may use to demonstrate compliance with the limits. The values for air concentrations are derived to assure that doses will be less than 50 mrem/yr. The revised rule retains the requirement for uranium fuel cycle facilities to comply with the standards set forth in 40 CFR 190.

# A.3.2 Appendix I to 10 CFR 50.

Appendix I to Part 50, "Numerical Guides for Design Objectives and Limiting Conditions for Operation to Meet the Criterion As Low as is Reasonably Achievable for Radioactive Material in Light-Water-Cooled Nuclear Power Reactor Effluents," establishes (a) numerical ALARA design objectives that apply during the design and licensing of a new facility and (b) limiting conditions of operation that apply to operating plants

The Appendix I design objectives require applicants for a permit to construct a nuclear power reactor to describe how radionuclide releases to unrestricted areas will be kept as low as is reasonably achievable. The applicant satisfies the design objectives, in part, by demonstrating that the gaseous radionuclide releases to the atmosphere from each light-water reactor on site will not result in an estimated average annual air dose in excess of 10 mrad for gamma exposure and 20 mrad for beta exposure. These limits, applicable to any unrestricted location that can be occupied, pertain only to external exposure to noble gases. Lower radionuclide release rates may be required to satisfy the design objectives if the releases are likely to result in an estimated annual external dose from gaseous effluents in excess of 5 mrem/yr. Alternatively, higher release rates may be acceptable if the applicant can provide reasonable assurance that the external dose to any individual in unrestricted areas will not exceed 5 mrem/yr to the whole body and 15 mrem/yr to the skin. The applicant must also demonstrate that the calculated annual total quantity of all radiolodines and radioactive particulates to be released to the atmosphere from each light-water reactor will not cause exposures to any individual in unrestricted areas in excess of 15 mrem to any organ.

There are no criteria regarding total whole-body dose from both external and internal exposures from radionuclides released to the atmosphere. The 5 and 15 mrem criteria are for external exposures to noble gases. Nevertheless, some nuclear utilities employ dose calculation procedures that include the calculation of internal whole-body dose using the whole-body dose conversion factors provided in the NRC's guidance (NRC 77a) and assess these doses against a 5 and 15 mrem/yr limit.

In addition to these numerical limits for individual exposures, Appendix I also requires the applicant to demonstrate that all cost-effective items of reasonably demonstrated technology have been incorporated into the radwaste system design, including effluent controls. A cost-effective technology is defined as any technology that can reduce the cumulative offsite dose by 1 person rem at a cost of no more than \$1,000.

To assist applicants in assessing and demonstrating compliance with the design objectives of Appendix I, the NRC's staff developed a series of regulatory guides providing acceptable methods for calculating the pre-operational estimates of effluent releases, dispersion of effluents in the environment, and associated doses to members of the public. The methodologies were selected to be "conservatively realistic", that is, to ensure that the doses are neither significantly underestimated nor unrealistically overestimated.

## (1) The Source Term

The first step in determining compliance with the Appendix I design objectives is the calculation of the anticipated radionuclide release rate during normal plant operations, including anticipated operational occurrences. To assist applicants with these calculations, the NRC's staff developed the PWR and BWR Gaseous and Liquid Effluent code, referred to as the PWR and BWR GALE code Detailed descriptions of the codes, along with their user's manuals and listing, are available (NRC 76a, b, and c)

The codes are designed to model the buildup of radionuclides in various systems and compartments in the plants and estimate the radionuclide release rates to the environment in liquid and caseous effluents during resmal operation, including anticipated operational occurrences. The code can model a broad range of plant parameters and radwaste systems designs.

In the 1970's, the off-gas treatment systems of most BWRs were redesigned to increase greatly the holdup time of noble gases, resulting in large reductions in effluents. Other modifications included the more widespread use of charcoal filtration systems on building exhausts to remove radioiodines, special provisions to control steam leakage from steam line valves, and the use of clean steam, as opposed to turbine steam, to limit air inleakage through the turbine shaft seal.

# (2) Atmospheric Dispersion and Deposition

Once the source term is estimated, in units of Ci/yr of individual radionuclides, the next step in the assessment process is modeling the atmospheric dispersion and deposition of radionuclides in the environment. The purpose of this calculation is to determine the radionuclide concentrations in air and on soil at actual and potential offsite receptor locations. To assist applicants with these calculations, the NRC's staff developed the computer code XOQDOQ The NRC's guidance pertaining to the use of this code is provided in references NRC 72, 77b, and 77c.

The code uses mateorological data obtained from an onsite meteorological tower to generate joint frequency data and calculates annual average atmospheric dispersion factors (X/Q values) and deposition factors (D/Q values). Together with the source term, the X/Q

and D/Q values are used to estimate the average annual radionuclide concentrations in air and on soil offsite

The design, installation, testing, and reliability of the meteorological tower used to obtain the input data to XOQDOQ are governed by NRC's quality assurance regulations. Specific guidance pertaining to the meteorological program is provided in Regulatory Guide 1.23 (NRC 72).

# (3) <u>Dose Calculations</u>

The final step in assessing compliance with the Appendix I design objectives is the calculation of the doses to members of the general public. To assist applicants with the gaseous effluent dose calculations, NRC's staff has provided the computer code GASPAR. A detailed description of the code and its users manual is provided in Regulatory Guide 1.109 (NRC 77a)

GASPAR uses the source terms from GALE and the atmospheric dispersion and deposition factors from XOQDOQ as input, along with site-specific demographic and land use factors, to calculate the annual average dose to real or hypothetical individuals at specified offsite locations (mrem/yr) and to the population within 50 miles of the site (person rem/yr).

The exposure pathways included in GASPAR are

- External exposure from the passing plume,
- Internal exposure from inhalation of airborne radionuclides,
- External exposure to deposited radionuclides, and
- Internal exposure from the ingestion of vegetables, beef, and milk contaminated as a result of deposited radionuclides.

The results of the mrem/yr calculations are used to assess compliance with the 5 and 15 mrem/yr design objectives of Appendix I for the maximally exposed individual. The result of the person-rem/yr calculation is used as input into the assessment of the \$1,000 per man rem criteria of Appendix I. To assist applicants in assessing compliance with the cost-benefit criteria, the NRC's staff issued Regulatory Guide 1.110 (NRC 76d). This guide provides equipment cost information and guidance on calculation of annualized cost, which is used, along with the annual person rem doses, to obtain a cost-benefit ratio.

Application of these codes after promulgation of Appendix I led to significant redesign and backfitting of improved effluent control systems at both PWRs and BWRs. For the older BWRs, the basic design of the gaseous radwaste treatment systems has changed considerably. The original design of the off-gas treatment systems for older BWRs was limited to a 30-minute delay line and often included a fully elevated release to enhance atmospheric dilution. The radionuclide effluents from these systems were relatively large but were acceptable under the standards applicable at that time. Modifications to the gaseous radwaste management systems of PWRs include:

- more extensive use of charcoal filtration on effluent release points,
- increased gas decay tank capacity,
- more widespread use of systems to eliminate gaseous emissions associated with steam generator blowdown,
- venting the gaseous emissions from the mechanical vacuum pump to the condenser, which virtually eliminates this source of radioiodine emission, and
- special provisions to control steam leakage from steam line valves.

The limiting conditions of operation (LCOs) set forth in Appendix I complement the design objectives by providing guidance to ensure that, during operation, the facility maintains radionuclide releases and offsite exposures ALARA and consistent with the design objectives At the same time, the LCOs provide for flexibility of operation, compatible with considerations of public health and safety, to ensure a dependable source of power even under unusual operating conditions.

Compliance with the LCOs is established through Radiological Effluent Technical Specifications (RETS) in accordance with Part 50.36a. The LCOs and their associated RETS require that, if the quantity of radioactive materials actually released in effluents to unrestricted areas in any calendar quarter is such that the resulting radiation exposure, calculated on the same basis as the design objectives, exceeds one-half the annual design objectives, the licensee is required to investigate the cause of the excess, define and initiate a program of corrective action, and report these actions to the NRC within 30 days from the end of the quarter during which the release occurred.

The LCOs and RETS also require the licensee to initiate effluent and environmental monitoring programs to provide (1) data on the quantities of radionuclides released, (2) the levels of radiation and radioactive materials in the environment, and (3) changes in land use and demography in the vicinity of the site that pertain to compliance with the LCOs. If the monitoring data reveal that the relationship between the quantities of radioactive materials released and the doses to individuals in unrestricted areas is significantly different than that assumed in the calculations used to assess compliance with the design objectives, the NRC's staff may require a modification of the RETS.

To assist licensees in complying with the LCOs and preparing their RETS, the NRC's staff issued several guidance documents (NRC 83a and b, NRC 78) In addition, NRC also provides guidance on periodic reporting requirements pertaining to the LCOs (NRC 74) These documents provide highly detailed standard RETS and procedures for implementing the RETS

Guidance is also provided on ensuring compliance with 40 CFR 190, which establishes limits on the total doses from all radionuclides (except radon) and all pathways of 25 mrem/yr to the whole body, 75 mrem/yr to the thyroid, and 25 mrem/yr to any organ. Accordingly, the Appendix I doses must be carefully summed to use them for assessing compliance with 40 CFR 190. In addition, since liquid and gaseous pathways must be summed, the maximum individuals may differ. Further, the limits in 40 CFR 190 are for exposure to the fuel cycle and not to a single power plant. This has significance for plants with more than one reactor. NUREG-0133 (NRC 78) provides guidance to utilities for calculating doses for the purpose of assessing compliance with 40 CFR 190.

A complete listing of the NRC's Regulatory Guides is presented in Appendix B

#### REFERENCES

- NRC 72 U.S Nuclear Regulatory Commission "Onsite Meteorological Programs." Regulatory Guide 1.23. March 1972
- NRC 74 U.S. Nuclear Regulatory Commission. "Measuring, Evaluating, and Reporting Radioactivity in Solid Waste and Releases of Radioactive Material in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants."

  Regulatory Guide 1.21, Revision 1. June 1974.
- NRC 76a
  U.S. Nuclear Reg: 'tory Commission "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-Water-Cooled Power Reactors" Regulatory Guide 1.112. April 1976.
- NRC 76b U.S Nuclear Regulator, Commission. "Calculation of Releases of Radioacure Materials in Gaseous and Liquid Effluents from Pressurized Water Reactors (PWR-GALE Code)" NUREG-0017. April 1976.
- NRC 76c
  U.S. Nuclear Regulatory Commission. "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Boiling Water Reactors (BWR-GALE Code)." NUREG-0016 April 1976
- NRC 76d U.S. Nuclear Regulatory Commission "Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors." Regulatory Guide 1.110. March 1976
- NRC 77a

  U.S. Nuclear Regulatory Commission. "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR 50, Appendix I." Regulatory Guide 1.109, Revision 1 October 1977
- NRC 77b

  U.S Nuclear Regulatory Commission. "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Power Reactors." Regulatory Guide 1.111, Revision 1 July 1977.
- NRC 77c

  U.S. Nuclear Regulatory Commission. "XOQDOQ, Program for the Meteorological Evaluation of Routine Effluent Releases at Nuclear Power Stations" Report 0324. September 1977.
- NRC 78 U.S. Nuclear Regulatory Commission. "Preparation of Radiological Effluent Technical Specifications for Nuclear Power Plants" NUREG-0133 October 1978

U.S. Nuclear Regulatory Commission "Standard Radiological Effluent Techni-NRC 83a cal Specifications for PWRs " NUREG-0472 January 1983 NRC 83b U.S Nuclear Regulatory Commission. "Standard Radiological Effluent Technical Specifications for BWRs." NUREG-0473. January 1983 NRC 89a U.S Nuclear Regulatory Commission. "The 1989 NRC Annual Report." NUREG-1145, Vol 6 July 1989 NRC 89b U.S. Nuclear Regulatory Commission. "John B Martin, NRC Regional Administrator, Region V, to Sacramento Municipal Utility District, Docket No. 50-312." EA 86-110. January 8, 1989 NRC 90b U.S Nuclear Regulatory Commission "U S Nuclear Regulatory Commission Functional Organization Charts " NUREG-0325, Revision 14 August 15, 1990

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# APPENDIX B

# SELECTED NRC REGULATORY GUIDES

This appendix provides a list of the regulatory guides published by the NRC Guides that are particularly relevant to airborne effluent are denoted by an asterisk and boldface type

NO.	TITLE	REV	DATE
	DIVISION 1 - POWER REACTORS		
11	Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal System Pumps	<u>-</u>	11/70
1.2	Thermal Shock to Reactor Pressure Vessels	•	11/70
13	Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss-of-Coolant Accident for Boiling Water Reactors	- 1 2	11/70 06/73 06/74
14	Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss-of-Coolant Accident for Pressurized Water Reactors	- 1 2	11/70 06/73 06/74
1 5	Assumptions Used for Evaluating the Potential Radiological Consequences of a Steam Line Break Accident for Boiling Water Reactors		03/71
1 6	Independence Between Redundant Standby (Onsite) Power Sources and Between Their Distribution Systems	•	03/71
17	Control of Combustible Gas Concentrations in Containment Following a Loss-of-Coolant Accident	- 1 2	03/71 09/76 11/78
1.8	Qualification and Training of Personnel for Nuclear Power Plants	- 1 1-R 2 -	03/71 09/75 05/77 04/87
1.9	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (Onsite) Electric Power Systems at Nuclear Power Plants	- 1 2	03/71 11/78 12/79
1 10	Withdrawn		07/81
1 11	Instrument Lines Penetrating Primary Containment Supplement to Safety Guide 11	-	03/71 02/72

1.12	Instrumentation for Earthquakes	1	03/71 04/74
1 13	Spent Fuel Storage Facility Design Basis	1	03/71 12/75
1 14	Reactor Coolant Pump Flywheel Integrity	- 1	10/71 08/75
1.15	Withdrawn		07/81
1.16	Reporting of Operating Information - Appendix A Technical Specifica- tions	1 2 3 4	10/71 10/73 09/74 01/75 08/75
1.17	Protection of Nuclear Power Plants Against Industrial Sabotage	1	10/71 06/73
1 18	Withdrawn		07/81
1.19	Withdrawn		07/81
1.20	Comprehensive Vibration Assessment Program for Reactor Internals During Pre-operational and Initial Startup Testing	1 2	12/71 06/75 05/76
1.21*	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gas- eous Effluents from Light-Water-Cooled Nuclear Power Plants	1	12/71 06/74
1.22	Periodic Testing of Protection System Actuation Functions	•	02/72
1.23*	Onsite Meteorological Programs	-	03/72
1.24	Assumptions Used for Evaluating the Potential Radiological Consequences of a Pressurized Water Reactor Gas Storage Tank Failure	-	03/72
1.25	Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors	•	03/72
1 26	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants	- 1 2 3	03/72 09/74 06/75 02/76
1.27	Ultimate Heat Sink for Nuclear Power Plants	- 1 2	03/72 03/74 01/76
1.28	Quality Assurance Program Requirements	- 1 2 3	06/72 03/78 02/79 08/85

1.29	Seismic Design Classification	-	06/72
		1	08/73
		2	02/76
		3	09/78
1 30	Quality Assurance Requirements for the Installation, Inspection, and Testing of Instrumentation and Electric Equipment	•	08/72
1 31	Control of Ferrite Content in Stainless Steel Weld Metal	-	08/72
l		1	06/73
		2	05/77
		3	04/78
1 32	Criteria for Safety-Related Electric Power Systems for Nuclear Power	_	08/72
1 32	Plants	1	03/76
	riants	-	
		2	02/77
1 33	Quality Assurance Program Requirements (Operation)	-	11/72
		1	02/77
		2	02/78
1 34	Control of Electroslag Weld Properties	-	12/72
1.35	Inservice Inspection of Ungrouted Tendons in Prestressed Concrete	-	02/73
	Containment Structures	1	06/74
		2	01/76
		3	08/90
	1.35 1 Determining Prestressing Forces for Inspection of Prestressed Concrete Containments	-	08/90
1 36	Nonmetallic Thermal Insulation for Austenitic Stainless Steel	-	02/73
1 37	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated Components of Water-Cooled Nuclear Power Plants	-	03/73
1 38	Quality Assurance Requirements for Packaging, Shipping, Receiving,	_	03/73
	Storage, and Handling of Items for Water-Cooled Nuclear Power Plants	1	10/76
		2	05/77
1.39	Housekeeping Requirements for Water-Cooled Nuclear Power Plants	_	03/73
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		2	09/77
1 40	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment of Water-Cooled Nuclear Power Plants	-	03/73
1 41	Pre-operational Testing of Redundant On-Site Electric Power Systems To Verify Proper Load Group Assignments	-	03/73
1 42	Withdrawn		03/76
1.42	Control of Stomless Stool Wolds Cladding of Law Allaw Stool Commo		05/73
1 43	Control of Stainless Steel Welds Cladding of Low-Alloy Steel Components		03/13
1 44	Control of the Usc of Sensitized Stainless Steel	-	05/73
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1 45	Reactor Coolant Pressure Boundary Leakage Detection Systems	_	05/73
1 46	Withdrawn		03/85
1 47	Bypassed and Inoperable Status Indication for Nuclear Plant Safety Systems	- -	05/73
1.48	Withdrawn		03/85
1 49	Power Levels of Nuclear Power Plants	1	05/73 12/73
1.50	Control of Preheat Temperature for Welding Low-Alloy Steel	•	05/73
1 51	Withdrawn		07/75
1 52	Design, Testing, and Maintenance Criteria for Postaccident Engineered- Safety-Feature Atmosphere Cleanup System Air Filtration and Adsorp- tion Units of Light-Water-Cooled Nuclear Fower Plants	1 2	06/73 07/76 03/78
1.53	Application of the Single-Failure Criterion to Nuclear Power Plant Protection Systems	-	06/73
1.54	Quality Assurance Requirements for Protective Coatings Applied to Water-Cooled Nuclear Power Plants	•	06/73
1.55	Withdrawn		07/81
1.56	Maintenance of Water Purity in Boiling Water Reactors	1	06/73 07/78
1 57	Design Limits and Loading Combinations for Metal Primary Reactor Containment System Components	•	06/73
1.58	Qualification of Nuclear Power Plant Inspection, Examination, and Testing Personnel	- 1	08/73 09/80
1.59	Design Basis Floods for Nuclear Power Plants	- 1 2	08/73 04/76 08/77
1.60	Design Response Spectra for Seismic Design of Nuclear Power Plants	- 1	10/73 12/73
1 61	Damping Values for Seismic Design of Nuclear Power Plants		10/73
1.62	Manual Initiation of Protective Actions	-	10/73
1.63	Electric Penetration Assemblies in Contamment Structures for Nuclear Power Plants	1 2 3	10/73 05/77 07/78 02/87
1 64	Quality Assurance Requirements for the Design of Nuclear Power Plants	1 2	10/73 02/75 06/76

1 65 1 66 1 67 1 68	Materials and Inspections for Reactor Vessel Closure Studs  Withdrawn  Withdrawn	<u>-</u>	11/73 10/77
1 67			10/77
	Withdrawn		
1 68			04/83
	Initial Test Programs for Water-Cooled Nuclear Power Plants	- 1 2	11/73 01/77 08/78
	1 68 I Pre-operational and Initial Startup Testing of Feedwater and Condensate Systems for Boiling Water Reactor Plants	1	12/75 01/77
	1 68 2 Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-Cooled Nuclear Power Plants	1	01/77 07/78
	1 68 3 Pre-operational Testing of Instrument and Control Air Systems	•	04/82
1 69	Concrete Radiation Shields for Nuclear Power Plants	-	12/73
1 70	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants	- 1 2 3	02/72 10/72 09/75 11/78
1 71	Welder Qualification for Areas of Limited Accessibility	•	12/73
1 72	Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin	- 1 2	12/73 01/78 11/78
1 73	Qualification Tests of Electric Valve Operators Installed Inside the Containment of Nuclear Power Plants	•	01/74
1 74	RESERVED		
1 75	Physical Independence of Electric Systems	1 2	02/74 01/75 09/78
1 76	Design Basis Tornado for Nuclear Power Plants	-	04/74
1 77	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized Water Reactors	•	05/74
1 78	Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release	-	06/74
1.79	Pre-operational Testing of Emergency Core Cooling Systems for Pressurized Water Reactors	1	06/74 09/75
1 80	Withdrawn		05/82
1 81	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power Plants	1	06/74 01/75

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1.86   Termination of Operating Licenses for Nuclear Reactors   - 06/74	1 84		27	
1.87 Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors  1 06/75  1 88 Collection, Storage, and Maintenance of Nuclear Power Plant Quality Assurance Records  1 12/75  1 89 Environmental Qualification of Certain Electric Equipment Important to Safety for Nuclear Power Plants  1 90 Inservice Inspection of Prestressed Concrete Containment Structures with Grouted Tendons  1 91 Evaluations of Explosions Postulated To Occur on Transportation Routes Near Nuclear Power Plants  1 92 Combining Modal Responses and Spatial Components in Seismic Response Analysis  1 93 Availability of Electric Power Sources  1 94 Quality Assurance Requirements for Installation, Inspection, and Testing of Structural Concrete and Structural Steel During the Construction Phase of Nuclear Power Plants  1 95 Protection of Nuclear Power Plant Control Room Operators Against an Accidental Chlorine Release  1 04/76  1 96 Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water Nuclear Power Plants  1 1 2/78  1 1 1 2/78  1 1 2/79 Instrumentation for Light-Water-Cooled Nuclear Power Plants To Assess Plant and Environs Conditions During and Following an Accident  1 1 2/75  1 2 2 3 05/83	1.85	Materials Code Case Acceptability - ASME Section III, Division 1	- 27	
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Accidental Chlorine Release 1 01/77  1.96 Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water Nuclear Power Plants 1 06/76  1.97 Instrumentation for Light-Water-Cooled Nuclear Power Plants To Assess Plant and Environs Conditions During and Following an Accident 2 12/80 3 05/83	1 94	of Structural Concrete and Structural Steel During the Construction		04/75 04/76
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Assess Plant and Environs Conditions During and Following an Accident 2 12/80 3 05/83	1.96		- 1	05/75 06/76
1 98 Assumptions Used for Evaluating the Potential Radiological Conse 03/76	1.97	Assess Plant and Environs Conditions During and Following an Acci-	2	12/75 08/77 12/80 05/83
quences of a Radioactive Offgas System Failure in a Boiling Water Reactor	1 98	, ,	-	03/76

1 99	Radiation Embrittlement of Reactor Vessel Materials	1 2	07/75 04/77 05/88
1 100	Seismic Qualification of Electric and Mechanical Equipment for Nuclear Power Plants	1 2	03/76 08/77 06/88
1 101	Emergency Planning and Preparedness for Nuclear Power Reactors	1 2	11/75 03/77 10/81
1 102	Flood Protection for Nuclear Power Plants	1	10/75 09/76
1 103	Withdrawn		07/81
1 104	Withdrawn		08/79
1 105	Instrument Setpoints for Safety-Related Systems	1 2	11/75 11/76 02/86
1 106	Thermal Overload Protection for Electric Motors on Motor-Operated Valves	1	11/75 03/77
1 107	Qualifications for Cement Grouting for Prestressing Tendons in Containment Structures	1	11/75 02/77
1 108	Periodic Testing of Diesel Generator Units as Onsite Electric Power Systems at Nuclear Power Plants	- 1	08/76 08/77
1.109*	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR 50, Appendix I	1	03/76 10/77
1.110*	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors	-	03/76
1.111*	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors	1	03/76 07/77
1 112	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-Water-Cooled Power Reactors	- O-R	04/76 05/77
1 113	Estimating Aquatic Dispersion of Effluents from Accidental and Rou- tine Reactor Releases for the Purpose of Implementing Appendix I	- 1	05/76 04/77
1 114	Guidance to Operators at the Controls and to Senior Operators in the Control Room of a Nuclear Power Unit	- 1 2	02/76 11/76 05/89
1 115	Protection Against Low-Trajectory Turbine Missiles	- 1	03/76 07/77

1 116	Quality Assurance Requirements for Installation, Inspection, and Testing of Mechanical Equipment and Systems	O-R	06/76 05/77
1.117	Tornado Design Classification	- 1	06/76 05/77
1.118	Periodic Testing of Electric Power and Protection Systems	1 2	06/76 11/77 06/78
1.119	Withdrawn		06/77
1 120	Fire Protection Guidelines for Nuclear Power Plants	1	06/76 11/77
1.121	Bases for Plugging Degraded PWR Steam Generator Tubes	-	08/76
1 122	Development of Floor Design Response Spectra for Seismic Design of Floor-Supported Equipment or Components	- 1	09/76 02/78
1 123	Quality Assurance Requirements for Control of Procurement of Items and Services for Nuclear Power Plants	- 1	10/76 07/77
1.124	Service Limits and Loading Combinations for Class 1 Linear-Type Component Supports	- 1	11/76 01/78
1 125	Physical Models for Design and Operation of Hydraulic Structures and Systems for Nuclear Power Plants	1	03/77 10/78
1 126	An Acceptable Model and Related Statistical Methods for the Analysis of Fuel Densification	1	03/77 03/78
1.127	Inspection of Water-Control Structures Associated with Nuclear Power Plants	- 1	04/77 03/78
1 128	Installation Design and Installation of Large Lead Storage Batteries for Nuclear Power Plants	- 1	04/77 10/78
1.129	Maintenance, Testing, and Replacement of Large Lead Storage Batteries for Nuclear Power Plants	- 1	04/77 02/78
1.130	Service Limits and Loading Combinations for Class 1 Plate-and-Shell- Type Component Supports	1	07/77 10/78
1.131	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-Water-Cooled Nuclear Power Plants	-	08/77
1 132	Site Investigations for Foundations of Nuclear Power Plants	- 1	09/77 03/79
1 133	Loose-Part Detection Program for the Primary System of Light-Water- Cooled Reactors	- 1	09/77 05/81
1.134	Medical Evaluation of Licensed Personnel for Nuclear Power Plants	- 1 2	09/77 03/79 04/87

1 135	Normal Water Level and Discharge at Nuclear Power Plants	-	09/77
1 136	Materials, Construction, and Testing of Concrete Containments	- 1 2	11/77 10/78 06/81
1 137	Fuel-Oil Systems for Standby Diesel Generators	1	01/78 10/79
1 138	Laboratory Investigations of Soils for Engineering Analysis and Design of Nuclear Power Plants	-	04/78
1.139	Guidance for Residual Heat Removal	-	05/78
1.140*	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System Air Filtration and Adsorption Units of Light- Water-Cooled Nuclear Power Plants	1	03/78 10/79
1 141	Containment Isolation Provisions for Fluid Systems	-	04/78
1 142	Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor Vessels and Containment)	- 1	04/78 10/81
1.143*	Design Guidance for Radioactive Waste Management Systems, Structures, and Components Installed in Light-Water-Cooled Nu- clear Power Plants	1	07/78 10/79
1 144	Auditing of Quality Assurance Programs for Nuclear Power Plants	1	01/79 09/80
1.145	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at Nuclear Power Plants	- 1	08/79 11/82
1.146	Qualification of Quality Assurance Program Audit Personnel for Nuclear Power Plants	•	08/80
1 147	Inservice Inspection Code Case Acceptability - ASME Section XI, Division 1	8	02/81 11/90
1 148	Functional Specification for Active Valve Assemblies in Systems Important to Safety in Nuclear Power Plants	•	03/81
1.149	Nuclear Power Plant Simulation Facilities for Use in Operator License Examinations	- 1	04/81 04/87
1 150	Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations	- 1	06/81 02/83
1 151	Instrument Sensing Lines	-	07/83
1 152	Criteria for Programmable Digital Computer System Software in Safety-Related Systems of Nuclear Power Plants	-	11/85
1 153	Criteria for Power, Instrumentation, and Control Portions of Safety Systems	-	12/85
	and the same of th	<u> </u>	

1 154	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis Reports for Pressurized Water Reactors	•	01/87
1 155	Station Blackout		08/88
1 156	Environmental Qualification of Connection Assemblies for Nuclear Power Plants	•	11/87
1.157	Best-Estimate Calculations of Emergency Core Cooling System Performance	•	05/89
1.158	Qualification of Safety-Related Lead Storage Batteries for Nuclear Power Plants	•	02/89
1 159	Assuring the Availability of Funds for Decommissioning Nuclear Reactors	•	09/90
	DIVISION 4 - ENVIRONMENTAL AND SITING		
4.1*	Programs for Monitoring Radioactivity in the Environs of Nuclear Power Plants	1	01/73 04/75
4.2	Preparation of Environmental Reports for Nuclear Power Stations	1 2	02/71 01/75 07/76
43	Withdrawn		12/76
44	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent Dispersion in Natural Water Bodies	•	05/74
4 5	Measurements of Radionuclides in the Environment - Sampling and Analysis of Plutonium in Soil	-	05/74
46	Measurements of Radionuclides in the Environment - Strontium-89 and Strontium-90 Analyses	-	05/74
47	General Site Suitability Criteria for Nuclear Power Stations	1	09/74 11/75
4.8*	Environmental Technical Specifications for Nuclear Power Plants	-	12/75
49	Preparation of Environmental Reports for Commercial Uranium Enrichment Facilities	1	12/74 10/75
4 10	Withdrawn		11/77
4 11	Terrestrial Environmental Studies for Nuclear Power Stations	1	07/76 08/77
4.12	(Not Published)		
4 13	Performance, Testing, and Procedural Specifications for Thermoluminescence Dosimetry Environmental Applications	- 1	11/76 07/77

4 14	Radiological Effluent and Monitoring at Uranium Mills	1	06/77 04/80
4.15*	Quality Assurance for Radiological Monitoring Programs (Normal Operations) - Effluent Streams and the Environment	- 1	12/77 02/79
4 16	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in Liquid and Gaseous Effluents from Nuclear Fuel Reprocessing and Fabrication Plants and Uranium Hexafluoride Production Plants	1	03/78 12/85
4 17	Standard Format and Content Guide of Site Characterization Plans for High-Level-Waste Geologic Repositories	- 1	07/82 03/87
4 18	Standard Format and Content of Environmental Reports for Near- Surface Disposal of Radioactive Waste	-	06/83
4 19	Guidance for Selecting Sites for Near-Surface Disposal of Low-Level Radioactive Waste	-	08/8
	DIVISION 8 - OCCUPATIONAL HEALTH		
8 1	Radiation Symbol	-	02/73
82	Guide for Administrative Practices in Radiation Monitoring		02/73
83	Film Badge Performance Criteria	-	02/73
34	Direct-Reading and Indirect-Reading Pocket Dosimeters	-	02/73
8 5	Criticality and Other Interior Evacuation Signals	- 1	02/73 03/81
86	Standard Test Procedure for Geiger-Muller Counters		05/73
87	Occupational Radiation Exposure Records System	-	05/73
8.8*	Information Relevant to Ensuring that Occupational Radiation Exposures at Nuclear Power Stations Will Be As Low As Is Reasonably Achievable	- 1 2 3	07/73 09/75 03/77 06/78
89	Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program	•	09/73
8.10*	Operating Philosophy for Maintaining Occupational Radiation Exposures As Low As Is Reasonably Achievable	- 1 1-R	04/74 09/75 05/77
8 11	Applications of Bioassay for Uranium	-	06/74
8 12	Criticality Accident Alarm Systems	1 2	12/74 01/81 10/88

8 13	Instruction Concerning Prenatal Radiation Exposure	- :	03/75
1		1 2	11/75
-			
8 14	Personnel Neutron Dosimeters	-	06/76 08/77
<b> </b>			V6///
8.15	Acceptable Programs for Respiratory Protection	-	10/76
8.16	(Not Issued)	-	-
8 17	(Not Issued)	-	-
8 18	Information Relevant to Ensuring that Occupational Radiation Exposures at Medical Institutions Will Be As Low As Reasonably Achievable	1	12/77 10/82
8.19	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants - Design Stage Man-Rem Estimates	- 1	05/78 06/79
8.20	Applications of Bioassay for I-125 and I-131	- 1	04/78 09/79
8.21	Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and Manufacturing Plants	- 1	05/78 10/79
8.22	Bioassay at Uranium Mills	- 1	07/78 08/88
8 23	Radiation Safety Surveys at Medical Institutions	- 1	02/79 01/81
8.24	Health Physics Surveys During Enriched Uranium-235 Processing and Fuel Fabrication	1	11/78 10/79
8 25	Calibration and Error Limits of Air Sampling Instruments for Total Volume of Air Sampled	-	08/80
8.26	Applications of Bioassay for Fission and Activation Products	-	09/80
8.27	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power Plants	•	03/81
8.28	Audible-Alarm Dosimeters		08/81
8.29	Instruction Concerning Risks from Occupational Radiation Exposure		07/81
8.30	Health Physics Surveys in Uranium Mills	-	06/83
8 31	Information Relevant to Ensuring that Occupational Radiation Exposures at Uranium Mills Will Be As Low As Reasonably Achievable	-	05/83
8.32	Criteria for Establishing a Tritium Bioassay Program		07/88

<sup>\*</sup> Applicable to power plant effluents

### APPENDIX C

### **CAP-88 INFORMATION SHEETS**

Pressurized Water Reactors (PWRs)

CAP-88 INFORMATION SHEET Source Category. Reactors Date: May 1991 Vogtle Units 1 and 2 Facility: Waynesboro, GA Address: VOGTLE Population Assessment JCL File Name: Array attached Latituda: <u>33</u>° 8 ° 81 • 45 • 52 \* Pop. File Name: VOGTLE Longitude: Individual Assessment JCL Flie Name. Distances (meters): 200\_\_\_ 400 500 600 800 1200 300 1600 4800 6400 8000 10,000 15,000 20,000 30,000 40,000 50,000 60,000 80,000 Food Fractions: F1 F2 F3 Meat ·442 .558 .000 Urban .399 .601 Milk .000 Rural Vea .700 .300 .000 WEATHER DATA: STAR File Name: \_\_\_\_\_AGS1018 \_\_\_\_ Array attached WBAN: 03820 Temperature \_\_\_\_16\_\_ ℃ 1018 Header. Lid Height meters Code. Rainfall 123 cm/vr Set No. STACK Source: 1 2 3 4 5 6 Height (m): 72 Diameter (m): \_ AREA Source Diameter (m<sup>3</sup>): BUOYANT cal/sec MOMENTUM m/s \_\_\_\_\_ X ENTERED Rise Pasquill Category:

COMMENTS: Augusta Met Data approximately 25 miles N of plant (5-year)

Nuclide

Class Diam Ci/vr

# File Name VOGTLE

### Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											
NNW									3		
NW									3		6
WNW									3	11	44
W					_				11	17	11
WSW							6				
SW					_				3	17	11
SSW											
S							•				
SSE											
SE											6
ESE											
E										_	
ENE											
NE											
NNE											

#### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N			-						
NNW									
NW	11		_						
WNW	38								
W	49								
WSW	33		-			•			
SW	49								
SSW	11								
s	21								
SSE	6			-					
SE	27								
ESE									
E									
ENE									
NE									
NNE									

INSTALLATION. VOGTLE 1 & 2 

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE. PWR LICENSEE: GEORGIA POWER

DOCKET NO : 50-424 LICENSED POWER (MWT): 3.41E+03 THERMAL POWER (MWH): 2.18E+07 NET ELECTRIC POWER (MWH): 6.79E+06 COMMERCIAL OPERATION: 06/01/87 INITIAL CRITICALITY 03/09/87

COOLING WATER SOURCE. SAVANNAH RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
BE-7	6.58E-06
AR-41	3.55E+01
CO-57	3 46E-08
CO-58	1 49E-06
KR-851	3 38E-04
KR-85M	1 39E-01
KR-88	3.79E-04
SR-90	9.94E-08
I-131	1.52E-05
XE-131M	3.08E-05
I-133	7 93E-05
XE-133	1 19E+02
XE-133M	5.44E-02
XE-135	1 13E+01

TOTAL AIRBORNE TRITIUM RELEASE 1 08E+02

CAP-88 INFORMATION SHEET Source Category: Reactors Date: \_\_april 1991 ... Arkanas Nuclear One, Units 1 & 2 Facility: Address: Russelville, AR Population Assessment JCL File Name: ARKANAS Array attached 35° 18' 36° 93° 13' 50° Pop. File Name: Latitude: ARKANSAS Longitude: Individual Assessment JCL File Name: Distances (meters) <u>200 300 400 500 600 800 1200 1600 3200 4800</u> 6400 8000 10000 15000 20000 30000 40000 50000 60000 80000 F1 F2 F3 Meat .008 .992 .000 Food Fractions: Urban Milk .000 1.00 .000 Rural Veg .076 .924 .000 WEATHER DATA: STAR File Name: \_\_ LIT0516 Array attached Temperature: 17 ℃ 13963 WBAN: Lid Height: 1050 meters Header: 125\_cm/yr Code: Ramfail: Set No: STACK Source: 3 1 2 4 6 Height (m): 55\_\_\_ Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec \_\_\_\_\_\_ MOMENTUM m/s \_\_\_\_\_ \_\_\_ ENTERED Rise Pasquill Category: Nuclide Class Diam Ci/vr

COMMENTS: Met data 1955-1964 Little Rock 60 miles SE or plant

File Name. ARKANSAS Distances

	200	300	400	500	600	800	1200	1600	3200	4800	6400
N								19	62	71	180
NNW								19	146	212	300
NW								3	585	850	888
WNW								3	_13	27	112
W								13	13	13	19
WSW								5	5	5	90
SW								5	5	19	112
SSW								13	13	32	32
s								13	13	13	69
SSE		,						5	5	5	13
SE								19	27	90	167
ESE										217	780
E										27	146
ENE								48	89	146	345
NE								19	81	222	246
NNE							3		98	162	210

	Distances											
_	8000	10000	15000	20000	30000	40000	50000	60000	80000			
N	230											
NNW	368											
NW	989											
WNW	168											
W	27											
WSW	485											
SW	132											
SSW	62											
s	260	)										
SSE	41											
SE	328	3										
ESE	2280	)										
E	160	)										
ENE	470	)										
NE	271											
NNE	230	)										

INSTALLATION: ARKANSAS ONE 1 LOCATION - 6-MI WNW RUSSELLVILLE, AR

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: ARKANSAS P&L

**DOCKET NO.: 50-313** LICENSED PWR (MWT): 2.57E+03 THERMAL PWR (MWH): 1.24E+07
COMMERCIAL OPERATION: 12/19/74
LICENSED PWR (MWH): 2.57E+03
NET ELECTRIC PWR (MWH): 3.95E+06
INITIAL CRITICALITY: 08/06/74

COOLING WATER SOURCE: DARDANELLE RESERVOIR

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
K-40	5.96E-06
CO-58	1.74E-05
CO-60	7.07E-06
KR-85	3.22E+00
KR-85M	3.70E+00
SR-89	8.89E-06
SR-90	9.34E-08
NB-97	1.01E-06
AG-110M	1.27E-06
SN-113	9.44E-07
I-131	8.13E-04
XE-131M	1.67E+00
I-132	4.19E-06
I-133	2.99E-05
XE-133	1.16E+03
XE-133M	1.01E+00
CS-134	1.73E-06
XE-135	8.01E+01
CS-137	1.76E-04

TOTAL AIRBORNE TRITIUM RELEASE 3.46E+00

N/A=NOT APPLICABLE

N/D=NOT DETECTED

N/R=NOT REPORTED

INSTALLATION: ARKANSAS ONE 2
LOCATION: 6 MI WNW RUSSELVILLE, AR

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR LICENSEE: ARKANSAS P&L

DOCKET NO.: 50-368 LICENSED PWR (MWT): 2.82E+03

THERMAL PWR (MWH): 1.58E+07 NET ELECTRIC PWR (MWH): 4.95E+06 COMMERCIAL OPERATION: 03/26/80 INITIAL CRITICALITY: 12/05/78

)

COOLING WATER SOURCE: DARDANELLE RESERVOIR

AIRBORNE EFFLUE. TS	
NUCLIDES RELEASED	ACTIVITY (CI
ALPHA	1.69E-06
AR-41	1.98E-02
CR-51	5.61E-06
CO-57	2.18E-07
CO-58	5.89E-05
CO-60	2.17E-05
KR-85	1.78E-02
KR-85M	2.45E-01
KR-87	1.15E-02
KR-88	8.67E-02
RB-88	2.96E-05
SR-89	1.43E-06
SR-90	2.92E-08
SR-92	1.74E-06
NB-95	3.58E-06
ZR-95	4.24E-06
NB-97	1.04E-06
ZR-97	8.09E-07
RU-103	3.47E-07
AG-110M	1.51E- <b>0</b> 5
SN-113	5.56E-07
I-131	2.65E-04
XE-131M	3.84E+01
I-132	2.06E-06
I-133	3.85E-05
XE-133	1.79E+03
XE-133M	2.25E+00
XE-135	3.30E+02
CS-137	2.81E-05
BI-214	2.84E-04
PB-214	1.37E-05
RA-226	2.36E-06
TH-228	1.08E-05

TOTAL AIRBORNE TRITIUM RELEASE 5.00E+00

N/A=NOT APPLICABLE

N/D=NOT DETECTED

N/R=NOT REPORTED

Date: April 1991	CAP-88 INFOF	IS NOITAMI S	HEET ource Cate	gory: Rea	ctors	
Facility: Bee ver	Velley, Ü					
· court.	ngport. PA					
Population Asset		JCL File Na	ame: .	BRAVER		
Latitude: 40 Longitude: 80	37' 18	Pop. File	Name: .	BEAVER		
Individual Assess	sment	JCL File Na	ame:			
Distances (meters)  200 300 40 6400 8000 100			300 <u>12</u>	200 1600 000 5000		800 0000
Mili	at _008 _99	0000		Urban Rural		
WEATHER DATA:	ched		ile Name:	AGC1210	)	
WBAN:       14762         Header:       1210         Code:       AGC         Set No:	Tem Lidi Rain	perature: Height: fall:	00 mete			
STACK Source: Height (m): Diameter (m):	1 	2	3	4	5 6	
AREA Source Diameter (m <sup>3</sup> ):						-
BUOYANT cal/s MOMENTUM m	sec					-
_X ENTERED Rise Pasquill Categor	y: 0 0 B	$-\frac{c}{0}$	0 E	0 0 F G	meters	
Nuclide Class Dia	m Ci/yr:					- - -

COMMENTS: 1-yr met a ta for Pittsburgh

### File Name BEAVER

and gilden to the gallet for Section declares on					Di	stance	S				
_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N								16	16	96	380
NNW								190	420	40	50
NW								330	5800	1200	340
WNW								4	32	16	344
W										24	48
WSW									8	140	48
SW									20	344	136
SSW								8	24	52	72
S				0					16	100	48
SSE									12	24	36
SE								16	48	52	48
ESE								8	80	60	132
E		<del>-</del>							32	400	160
ENE								40	68	60	152
NE					_108				144	490	1100
NNE									320	180	180

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	104								
NNW	270								
NW	540								
WNW	600								
W	58								
WSW	88							•	
SW	44								
SSW	40								
s	132								
SSE	136								
SE	44								
ESE	200								
E	32	-							
ENE	88								
NE	266								
NNE	144								

INSTALLATION: BEAVER VALLEY LOCATION: SHIPPINGPORT, PA

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: DUQUESNE LIGHT
DOCKET NO.: 50-334 LICENSED PWR (MWT): 2.66E+03
THERMAL PWR (MWH): 1.64E+07 NET ELECTRIC PWR (MWH): 4.98E+06
COMMERCIAL OPERATION: 10/01/76 INITIAL CRITICALITY: 05/10/76

COOLING WATER SOURCE: OHIO RIVER

UNIT NUMBER: 2 TYPE: PWR LICENSEE: DUQUESNE LIGHT

DOCKET NO.: 50-412 LICENSED PWR (MWT): 2.66E+03

THERMAL PWR (MWH): 2.13E+07 NET ELECTRIC PWR (MWH): 6.48E+06 COMMERCIAL OPERATION: 11/17/87 INITIAL CRITICALITY: 08/04/87

COOLING WATER SOURCE: OHIO RIVER

### AIRBORNE EFFLUENTS

NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	3.02E-01
MN-54	1.46E-07
CO-58	1.26E-05
CO-60	1.25E-04
KR-85	4.72E-01
KR-85M	2.43E-06
SR-90	9.73E-09
MO-99	2.79E-08
I-131	1.56E-04
XE-131M	1.79E-01
I-133	1.59E-04
XE-133	2.32E+00
I-135	2.13E-05
XE-135	4.44E-01
XE-135M	2.35E-02
CS-137	1.37E-06

TOTAL AIRBORNE TRITIUM RELEASE 4.33E+01

INSTALLATION: BEAVER VALLEY 1 LOCATION: SHIPPINGPORT, PA

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: DUQUESNE LIGHT

LICENSED PWR (MWT): 2.66E+03 DOCKET NO.: 50-334

COMMERCIAL OPERATION: 10/01/76
COOLING WATER COUNCES THERMAL PWR (MWH): 1.64E+07 NET ELECTRIC PWR (MWH): 4.98E+06

INITIAL CRITICALITY: 05/10/76

COOLING WATER SOURCE: OHIO RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
CO-58	9.13E-04
CO-60	4.05E-04
KR-85	5.00E+01
I-131	1.70E-03
XE-131M	3.73E+00
I-133	1.16E-05
XE-133	2.03E+01
XE-133M	2.90E-02
CS-134	2.05E-06
XE-135	1.42E+00
CS-137	8.83E-05
CE-141	4.60E-06

INSTALLATION: BEAVER VALLEY 2 LOCATION: SHIPPINGPORT, = PA

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR LICENSEE: DUQUESNE LIGHT

DOCKET NO.: 50-412 LICENSED PWR (MWT): 2.66E+03
THERMAL PWR (MWH): 2.13E+07 NET ELECTRIC PWR (MWH): 6.48E+06
COMMERCIAL OPERATION: 11/17/87 INITIAL CRITICALITY: 08/04/87

COOLING WATER SOURCE: OHIO RIVER

AIRBORNE	<b>EFFLUENTS</b>	
NUCLIDES	RELEASED	ACTIVITY (CI)
CO-58		3.12E-05
CO-60		7.48E-05
KR-85		1.30E+01
MO-99		8.45E-07
I-131		1.18E-05
I-133		7.00E-06
XE-133		5.39E-01
XE-135		1.53E+00
CE-141		3.83E-07
CE-144		7.73E-06

Date: Apri	1 1991	2-88 INFO	RMATION	Spurce	Category	Reac	tors	
Facility:	Braidwood	Station	, Unit	1 & 2				<del>- i</del>
	Brandwood ation Assessm					********		<b>-</b>
	Array attached	d				ATDWOO_		_
Lautud	de: 41 tude: 88	16 <u>5</u>	Pop. F	ile Name	BR	AIDWOO		_
Individ	dual Assessme	nt	JCL File	Name:				_
Distances (n 200 30 6400 800	neters) 200 400 20 10000	_500 15000	600 20000	800 - 30000	1200 40000	1600 50000	3200 60000	4800 80000
Food Fraction		F1 F2			Urba	•		
	Milk	442 - 5 399 - 6 70 - 3	01000		Rura			
WEATHER [					me: ORD	0452		
WBAN: Header: Code: Set No:	94846	Lid	nperature Height: nfail:	. 950	meters cm/yr			
Heigh	K Source: it (m): eter (m):	1 61 2.8	2	3	4	5	_	
	Source eter (m²):				. <u></u>			<del></del>
BUOY	ANT cal/sec ENTUM m/s							
ENTE	RED Rise uill Category:	O B	<u>C</u>	<u>0</u> -	0 0 <b>E F</b>	<u>G</u> '	neters	
Nuclide 	Class Diam	Ci/yr·						
COMMENTS	S' Net deta	Chicag	o/ohare	50 ml	les NE	of plant	— —— t	

### File Name BRAIDWOO

Distances 300 200 400 **5**00 1200 1600 3200 4800 600 900 6400 N 3 950 NNW 3 1012 NW 3 WNW 1592 158 2283 0 572 3 W WSW 3 1239 322 842 SW 3 SSW 3 S 193 26 SSE SE **ESE** 1101 E 3 ENE NE 331 NNE 3

#### Distances

	8000	10000	15000	20000	30000	40000	50000	60000	80000
N				_					
NNW									
NW	571								
WNW	1720								
W									
WSW									
SW	721								
SSW									
S	•								
SSE									
SE									
ESE									
E	1817								
ENE	1930								
NE	2331								
NNE									

INSTALLATION: BRAIDWOOD 1
LOCATION: 24 MI SSW OF JOLIET, IL

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: COMMONWEALTH EDISON DOCKET NO.: 50-456 LICENSED PWR (MWT): 3.41E+03 THERMAL PWR (MWH): 1.04E+07 NET ELECTRIC PWR (MWH): 3.42E+06 COMMERCIAL OPERATION: 07/29/88 INITIAL CRITICALITY: 05/29/87

COOLING WATER SOURCE: KANKAKEE RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	3.31E-01
CO-58	1.20E-07
CO-60	1.90E-07
KR-85	1.78E-01
KR-85M	1.87E-02
KR-87	6.38E-04
KR-88	1.63E-02
I-131	3.22E-04
XE-131M	5.33E-01
I-133	1.27E-04
XE-133	4.02E+01
XE-133M	2.70E-01
I-134	2.40E-02
XE-135	3.81E-01

TOTAL AIRBORNE TRITIUM RELEASE 2.73E+00

INSTALLATION: BRAIDWOOD 2

LOCATION: 24 MI SSW OF JOLIER, III

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR
DOCKET NO.: 50-457
THERMAL PWR (MWH): 4.08E+06
COMMERCIAL OPERATION: 10/17/88
LICENSEE: COMMONWEALTH EDISON
LICENSED PWR (MWT): 3.41E+03
NET ELECTRIC PWR (MWH): 1.35E+06
INITIAL CRITICALITY: 03/08/88

COOLING WATER SOURCE: KANKAKEE RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	7.84E-01
KR-85	1.45E-02
KR-85M	1.19E-02
KR-87	9.60E-05
KR-88	5.66E-04
I-131	7.61E-05
XE-131M	8.85E-01
I-133	1.58E-05
XE-133	3.65E+01
XE-133M	1.09E-01
I-135	3.26E-06
XE-135	3.73E-02

TOTAL AIRBORNE TRITIUM RELEASE 2.66E+00

### CAP-88 INFORMATION SHEET

Date:	37 3.44		Source-Ca	tegory:	ke-ctor	<u>s</u>	<b>-</b>		
racinty.	ron Station J	nits 1 8	k 2				-		
Population Assessment JCL File Name: BYRON  Array attached Latitude: 42 7 48 Longitude: 89 15 17 Pop. File Name: BYRON									
Individual As	sessment	JCL File	Name:			<del></del>	-		
Distances (meters) 200 300 6400 8000	400 500 10000 15000	600 20000		1200 000	1600 50000	3200 60000	4800 80000		
Food Fractions:	F1 F2 Meat .008 .99 Milk .000 1.0 Veg .076 .92	0 000		Urban Rural					
WEATHER DATA: Array	attached		R File Name	. MLIC	269		-		
WBAN: 149 Header: 269 Code: MLI Set No:	9 Lid	nperature. Height: nfall:	<u>950</u> me	eters n/yr					
— STACK Sour Height (m): Diameter (m)	61	2	3	4	5	6	<u>-</u>		
AREA Sour Diameter (m						<del></del>			
BUOYANT C	al/sec						<u> </u>		
X ENTERED R Pasquill Cate	ise O O	<u>0</u> <u>C</u>	0 <u>0</u> E	<u>0</u> F	O me	ters			
Nuclide Class	Diam: Cı/yr:						<del></del>		

COMMENTS: Moline/waa Cities met a to 80 miles S4 of plant

File Name: BYRON

	- Distances										
_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N									4_	18	87_
NNW									11_	11	74
NW									250	124	18_
WNW								4	4	7	18
W							0_		7	45	15
WSW								4	22	5 <u>6</u>	60
SW		-						11	15	26	55
SSW							26		18	7	43
S							4		7_	19	16
SSE								8	31	8	45
SE									24	16	20
ESE								4	12	32	28
E			0						16	20	41
ENE									20	41	28
NE									28	49	53
NNE								4	4	45	1181

### Distances

	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	<sup>-</sup> 796	-							
NNW	12								
NW	40								
WNW	4								
W	34								
WSW	45								
SW	1223								
SSW	1231								
S	159								
SSE	20								
SE	49								
ESE	17								
E	28								
ENE	178								
NE	106								
NNE	766								

INSTALLATION: BYRON

LOCATION: 3 MI SW BYRON, IL

#### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-454

THERMAL PWR (MWH): 1.99E+07

COMMERCIAL OPERATION: 09/16/85

COOLING WATER SOURCE: ROCK RIVER

UNIT NUMBER: 2 TYPE: PWR

DOCKET NO.: 50-455

THERMAL PWR (MWH): 2.04E+07

COMMERCIAL OPERATION: 08/21/87

COOLING WATER SOURCE: ROCK RIVER

LICENSEE: COMMONWEALTH EDISON LICENSED PWR (MWT): 3.41E+03 NET ELECTRIC PWR (MWH): 6.29E+06

INITIAL CRITICALITY: 02/02/85

LICENSEE: COMMONWEALTH EDISON

LICENSED PWR (MWT): 3.41E+03

NET ELECTRIC PWR (MWH): 6.36E+06

INITIAL CRITICALITY: 01/09/87

### AIRBORNE EFFLUENTS

NUCLIDES RELEASED	,
AR-41	
KR-85	
KR-85M	
KR-87	
KR-88	
I-131	
XE-131M	
I-132	
I-133	
XE-133	
XE-133M	
I-134	
I-135	

1.06E+01 3.97E-01 5.57E-03 2.53E-01 1.28E-02 6.72E+00 4.45E-04 3.72E-03 1.73E+03 1.25E+01 2.52E-05 1.12E-03 1.71E+01 2.32E-05

1.78E-05

ACTIVITY (CI) 6.57E-01

TOTAL AIRBORNE TRITIUM RELEASE 1.62E+00

N/A=NOT APPLICABLE

N/D=NOT DETECTED

XE-135

CS-137

BA-LA-140

N/R=NOT REPORTED

Date:	A	pril l	<b>CA</b> 1 991	P-88 INF(	ORMATIO	Source,	Category	Resc	tors		
Facility Addre			llaway lton,		Unit 1					_	
	Pop	_ Array	attache	d 45 • 42	JCL File			LLAWAY			
	Ind	ividual A	ssessme	ent	JCL File	Name:					
200		(meters 300	400	500	600		1200	1600 50000	3200 60000	4800 80000	
•		ctions:	Meat Milk	F1 F2 .008 .9			Urba Rura				
WEAT	HEI	R DATA: Array					ıme:C	GM0670			
WBAN Heade Code: Set N	er:		25 70 GM	Lic	emperature d Height: ainfall:	13 .900 86	meters cm/yr				
	Hei	ACK Solight (m): meter (r		1 64	2	3	4	5		5 	
		EA Sou meter (r									
	BU	OYANT MENTU	cal/sec IM m/s								
<u>x</u>	EN Pa	ITERED squill Ca	Rise itegory:	<u>0</u> <u>4</u>	0 0 B C	<u>0</u> -	0 0 E F	<u>G</u>	meters		
Nuclid	_		BIFY L		NG Met		-yr Col	unbus,	ms 80 1	miles 3	of nl-

File Name: CALLAWAY

NNE

Distances 1200 1600 3200 NNW NW WNW W WSW SW SSW 5\_ S SSE SE ESE E ENE NE 

#### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW	87								
NW	36								
WNW	87								
W	43								
WSW	33								
SW	24					<del> </del>		<del></del>	
SSW	55								
S	7								
SSE	2								
SE	69								
ESE	20								
E	2								
ENE	7								
NE	32								
NNE	4				<del>,</del>				

INSTALLATION: CALLAWAY

LOCATION: 10 MI SE FULTON, MO

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: UNION ELECTRIC DOCKET NO.: 50-483 LICENSED PWR (MWT): 3.57E+03

THERMAL PWR (MWH): 2.76E+07 NET ELECTRIC PWR (MWH): 8.94E+06 COMMERCIAL OPERATION: 12/19/84 INITIAL CRITICALITY: 10/02/84

COOLING WATER SOURCE: MISSOURI RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
ALPHA	2.38E-06
AR-41	6.10E-01
KR-85	8.20E+01
KR-85M	? 50E+00
KR-88	7.28E-01
RB-88	8.30E-02
I-131	1.87E-05
XE-131M	2.40E+00
XE-133	5.60E+02
XE-133M	2.56E+00
XE-135	3.73E+01

TOTAL AIRBORNE TRITIUM RELEASE 1.53E+01

CAP-88 INFORMATION SHEET Source Category: Re ctors Date: Atril 1991 Calvert Cliffs, Units 1 & 2 Facility: Lusby, MD Address: CALVERT Population Assessment JCL File Name: Array attached 38 • 26 • 4 • Latitude: Longitude: 76 26 30 Pop. File Name: CALVERT Individual Assessment JCL File Name: Distances (meters) 200 300 400 500 600 800 1200 1600 3200 4800 6400 8000 10000 15000 20000 30000 40000 50000 60000 80000 Food Fractions: F1 F2 Meat .442 .558 .000 Urban Milk .399 .601 .000 Rural Veg \_700 \_300 \_000 WEATHER DATA: STAR File Name: NHKL 306 \_\_ Array attached Temperatura: 12 13721 WBAN: 1306 Header: Lid Height: \_\_600\_ meters Code: Rainfall: 99 cm/yr Set No: STACK Source: 1 2 3 5 Height (m): 48 Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec \_\_\_\_\_ \_\_\_ MOMENTUM m/s B C D E F G Y ENTERED Rise Pasquill Category: A

COMMENTS. Net data Paxuxent River 1-yr

Class Diam Ci/vr:

Nuclide

File Name: CALVERT 3

	Distances										
	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											
NNW											
NW									3	684	129
MNW									9	186	40
₩									44	4	49
WSW									27	18	44
SW									44	18	71
SSW									36	18	93
S									40	58	16
SSE								85	9	27	142
SE		•						85	140		
ESE											
Ε											
ENE											
NE											
NNE											

	Distances									
_	8000	10000	15000	20000	30000	40000	50000	60000 80000		
. K										
NNW										
NW	4									
MNM	92									
W	67									
WSW	60									
SW	18									
SSW	58									
S	160									
SSE	62									
SE	497									
ESE										
E										
ENE										
NE										
NNE										

INSTALLATION: CALVERT CLIFFS

· · LOCATION: 45 MI SE WASHINGTON, D.C.

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: BALTIMORE G&E

DOCKET NO.: 50-317 LICENSED PWR (MWT): 2.70E+03
THERMAL PWR (MWH): 1.62E+07 NET ELECTRIC PWR (MWH): 5.16E+06
COMMERCIAL OPERATION: 05/08/75 INITIAL CRITICALITY: 10/07/74

COOLING WATER SOURCE: CHESAPEAKE BAY

UNIT NUMBER: 2 TYPE: PWR LICENSEE: BALTIMORE G&E

DOCKET NO.: 50-318 LICENSED PWR (MWT): 2.70E+03

THERMAL PWR (MWH): 2.07E+07

COMMERCIAL OPERATION: 04/01/77

NET ELECTRIC PWR (MWH): 6.60E+06

INITIAL CRITICALITY: 11/30/76

COOLING WATER SOURCE: CHESAPEAKE BAY

## AIRBORNE EFFLUENTS

WINDOWNE ELLPOEMIS	
NUCLIDES RELEASED	ACTIVITY (CI)
BE-7	5.46E-06
AR-41	6.08E-02
CO-57	4.97E-07
CO-58	2.10E-05
CO-60	2.80E-06
KR-85	3.14E+02
KR-85M	4.46E+01
KR-87	9.08E+00
KR-88	4.46E+00
I-131	1.25E-01
XE-131M	2.55E+01
TE-132	6.82E-07
I-133	1.17E-01
XE-133	4.88E+03
XE-133M	3.89E+01
CS-134	2.04E-05
I-135	8.97E-05
XE-135	3.74E+02
XE-135M	3.26E-03
CS-137	1.17E-02
CE-144	9.86E-06

TOTAL AIRBORNE TRITIUM RELEASE 3.13E+01

CAP-88 INFORMATION SHEET Date: \_\_April 1991 Source Category: Catawba Nuclear Station. Units 1 & 2 Facility: -Lake Wylle, SC Address: CATAWBA Population Assessment JCL File Name: Array attached 34 • 59 • 41 • Latitude: 81 • 14 • 42 • Pop. File Name: CATAWBA Longitude: Individual Assessment JCL File Name: Distances (meters) 1600 800 1200 400 600 500 300 200 3200 4800 6400 8000 10000 15000 20000 30000 40000 50000 60000 80000 Food Fractions: F1 F2 F3 Meat .008 .992 .000 Urban Miik .000 1.000 .000 Rural Veg .076 .924 .000 WEATHER DATA: STAR File Name: CLT0682 Array attached WBAN: 13881 . Temperature: 37 °C 682 Lid Height: 1000 meters Header: 110 Code: Rainfall: \_cm/yr Set No: STACK Source: 2 3 1 4 5 6 Height (m): 38 Diameter (m): AREA Source Diameter (m<sup>\*</sup>): BUOYANT cal/sec \_\_\_\_\_ \_\_\_ \_\_\_ MOMENTUM m/s \_\_X\_ ENTERED Rise 0 0 0 0 0 0 meters

A B C D E F G Pasquill Category:

COMMENTS: Met data 5-yr Charlotte, NC 20 miles NE of plant

Nuclide

Class Diam Ci/yr

## File Name: CATAWBA

## Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N							10		65	190	1750
NNW								4	44	27	138
NW									88	31	48
WNW									87	28	86
W								6	<u>26</u>	31	74
WSW								3	20	50	48
SW						0	12		20	82	66
SS₩								10	55	231	286
S								22	33_	121	330
SSE							7		33	330	451
SE									37	558	997
ESE								24	115	1556	1298
E							3		30	334	434
ENE							4		60	30	130
NE							_7		90	81	150
NNE							7		48	301	578

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	825								
NNW	110								
NW	<b>27</b> 5								
WNW	97								
W	98								
WSW	55								
SW	110								
SSW	347								
S	440								
SSE	782								
SE	443								
ESE	130								
E	362								
ENE	200								
NE	177								
NNE	590								

LOCATION: 6 MI NNW OF ROCK HILL, SC

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: DUKE PWR CO
DOCKET NO.: 50-413 LICENSED PWR (MWT): 3.41E+03
THERMAL PWR (MWH): 2.29E+07 NET ELECTRIC PWR (MWH): 7.63E+06
COMMERCIAL OPERATION: 06/29/85 INITIAL CRITICALITY: 01/07/85

COOLING WATER SOURCE: LAKE WYLIE

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
F-18	3.18E-03
NA-24	1.38E-06
AR-41	5.99E+00
ĆR-51	6.34E-07
MN-54	2.85E-06
MN-56	7.97E-08
CO-58	1.52E-05
CO-60	6.81E-06
BR-80M	1.06E-06
BR-82	1.28E-06
KR-85	2.08E+00
KR-85M	9.23E-01
KR-87	1.31E-01
KR-88	1.03E+00
RB-88	7.96E-05
RB-89	1.57E-08
SR-90	3.26E-07
NB-95	7.70E-08
ZR-95	4.15E-10
TC-99M	3.79E-09
RU-106	3.53E-08
AG-108M	2.38E-09
SB-126	1.04E-08
I-131	7.62E-04
TE-131M	1.09E-08
XE-131M	1.22E+01
I-132	2.30E-06
I-133	4.50E-04
XE-133	1.51E+03
XE-133M	1.55E+01
CS-134	3.41E-08
I-134	1.44E-06
I-135	1.88E-06
XE-135	1.80E+01
XE-135M	5.68E-03
CS-137	5.74E-08
CS-138	1.13E-06
XE-138	3.60E-03
BA-139	4.72E-08
BA-140	8.32E-10

W-187 5.26E-08

TOTAL AIRBORNE TRITIUM RELEASE 3.03E+01

LOCATION: 6 MI NNW OF ROCK HILL, SC

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR

LICENSEE: DUKE PWR CO **DOCKET NO.: 50-414** LICENSED PWR (MWT): 3.41E+03

THERMAL PWR (MWH): 1.88E+07 NET ELECTRIC PWR (MWH): 6.17E+06

COMMERCIAL OPERATION: 08/19/86 INITIAL CRITICALITY: 05/08/86

COOLING WATER SOURCE: LAKE WYLIE

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
F-18	3.18E-03
NA-24	1.38E-06
AR-41	5.99E+00
CR-51	6.34E-07
MN-54	2.85E-06
MN-56	7.97E-08
CO-58	1.52E-05
CO-60	6.81E-06
BR-80M	1.06E-06
BR-82	1.28E-06
KR-85	2.08E+00
KR-85M	9.23E-01
KR-87	1.31E-01
KR-88	1.03E+00
RB-88	7.96E-05
RB-89	1.57E-08
SR-90	3.26E-07
NB-95	7.70E-08
ZR-95	4.15E-10
TC-99M	3.79E-09
RU-106	3.53E-08
AG-108M	2.38E-09
SB-126	1.04E-08
I-131	7.62E-04
TE-131M	1.09E-08
XE-131M	1.22E+01
I-132	2.30E-06
I-133	4.50E-04
XE-133	1.51E+03
XE-133M	1.55E+01
CS-134	3.41E-08
I-134	1.44E-06
I-135	1.88E-06
XE-135	1.80E+01
XE-135M	5.68E-03
CS-137	5.74E-08
CS-138 XE-138	1.13E-06 3.60E-03
BA-139	4.72E-08
BA-139 BA-140	
DH-140	8.32E-10

W-187 5.26E-08

TOTAL AIRBORNE TRITIUM RELEASE 3.03E+01

N/A=NOT APPLICABLE N/D=NOT DETECTED

N/R=NOT REPORTED

## CAP-88 INFORMATION SHEET

Date:	_ April_1991 _ Source Ca	ategory:	Reactors		
	Population Assessment JCL File Name:	CRYS	PAL		
	Address: Red River, FL				
—	Individual Assessment JCL File Name:		<del></del>	· · · · · · · · · · · · · · · · · · ·	
200	300 400 500 600 800 1				
	Meat <u>008 992 000</u> Milk <u>000 1.00 000</u> Veg <u>.076 .924 .000</u>	-			
WEAT	HER DATA: Array attached STAR File Name	e: TPAO	662		
Heade:	er: 662 Lid Height: 1100 m TPA Rainfall: 134 cm	eters			
—	Height (m):	4	5	6	
	BUOYANT cal/sec				
<u> </u>	ENTERED Rise 0 0 0 0 0 Pasquill Category: A B C D E	<u>0</u> -	0 mete	ers	
Nuclid	de Class Diam Ci/yr:				
COM	MENTS: Tampa met data 5-yr 70 miles S o	of plant			

C-34

File Name: CRYSTAL

Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											
NNW											
NW											
MNM											
W											
WSW											
SW											
SSW											
S											
SSE											3
SE											
ESE											
E											50
ENE											20
NE									0		10
NNE		<del></del>								\	3

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	40								
NNW	40								
NW									
WNW									
W									
WSW									
SW									
SSW									
S									
SSE									
SE									
ESE	500								
E	75		.,						
ENE	100								
NE	35								
NNE	30								

INSTALLATION: CRYSTAL RIVER -- -

LOCATION: 70 MI N TAMPA, FL

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 3 TYPE: PWR LICENSEE: FLORIDA PWR

LICENSED PWR (MWT): 2.45E+03 DOCKET NO.: 50-302

THERMAL PWR (MWH): 1.78E+07

COMMERCIAL OPERATION: 03/13/77

LICENSED PWR (MWT): 2.43E+03

NET ELECTRIC PWR (MWH): 5.77E+06

INITIAL CRITICALITY: 01/14/77

COOLING WATER SOURCE: GULF OF MEXICO

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
KR-85	2.45E+01
KR-85M	6.88E+00
SR-89	6.78E-07
SR-90	2.69E-07
I-131	9.87E-04
XE-131M	4.29E+01
I-133	7.27E-05
XE-133	3.26E+03
XE-133M	4.05E+00
CS-134	2.56E-05
XE-135	7.22E+01
CS-137	5.08E-05
CS-138	2.96E-07
UNIDENTIFIED	1.40E-04

#### TOTAL AIRBORNE TRITIUM RELEASE 9.93E+00

N/A=NOT APPLICABLE N/D=NOT DETECTED

N/R=NOT REPORTED

Date: Ap	ril 1991	IP-88 INFOR	MATION	SHEET Source	Category	Re ctor	s	_
Facility:	Davis-B	essie Uni	t 1					
Address:	Oak Har	bor. OH						•
	ulation Assessr Array attach		JCL File	Name:	DA	ris	<del></del>	•
Latit		35° 49°	Pop. Fi	le Name	): <u>Da</u>	/IS	······································	-
Indiv	vidual Assessm	ent	JCL File	Name:	<del></del>	<del></del>	<del></del>	•
	(meters) 300 400 8000 10000		600 20000	800 2000	1200 40000	1600 50000	3200 4 60000 8	
Food Fract	Meat Milk	F1 F2 .008 _99 .000 1.00	12 <u>.00</u> 0 .000		Urba Rura			
WEATHER				R File N	ате. <u>т</u>	07.0290		•
WBAN: Header: Code: Set No:	94830 990 TOL	Tem Lid I Rain	perature: Height: fall:	13 ,900 90	_ C _ meters _ cm/yr			
Heiç	CK Source: ght (m): neter (m):	76	2	3	4	5	6	 
	EA Source neter (m²):							<del></del>
BUO	DYANT cal/sec MENTUM m/s							_
_X_ ENT	TERED Rise quill Category:	0 0 A B	- °	0 -	0 0 E F	G m	eters	
Nuclide	Class Diam	Ci/yr:						- - -
COMMEN	TS: Tole	do 5 year r	ret data,	approx	cimately 2	5 miles NW	of the p	olant.

File Name: DAVIS

	200	300	400	500	600	800	1200	1600	3200	4800	6400
N							11				
NNW								67	19		
NM								4	149	43	
WNW									33	169	0
W							19		62	36	57
WSW									10	10	43
SW								16		35	59
SSW							21		43	43	64
5								9	47	71	_59_
SSE									47	52	85_
SE											
ESE											
E											
ENE					<del></del>						
NE							4				
NNE							30				

	Distances									
_	8000	10000	15000	20000	30000	40000	50000	60000	80000	
N										
NNW										
NW										
WNW										
W	91									
WSW	60									
SW	65									
SSW	92									
S	89									
SSE	81									
SE	107									
ESE										
Ξ										
ENE										
NE										
NNE										

INSTALLATION: DAVIS-BESSE LOCATION: 21 MI E TOLEDO, OH

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: TOLEDO EDISON &

CLEVELAND ELEC. FLLUM.

DOCKET NO.: 50-346

LICENSED PWR (MWT): 2.77E+03

THERMAL PWR (MWH): 3.91E+06 NET ELECTRIC PWR (MWH): 1.16E+06 COMMERCIAL OPERATION: 07/31/78 INITIAL CRITICALITY: 08/12/77

COOLING WATER SOURCE: LAKE ERIE

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
KR-85	4.94E+00
KR-88	9.96E-01
SR-89	₹8.22E-07
SR-90	<5.12E-07
I-131	4.76E-04
XE-131M	1.93E-02
I-133	1.64E-04
XE-133	1.02E+02
XE-133M	1.59E-01
XE-135	1.12E+00

TOTAL AIRBORNE TRITIUM RELEASE 4.99E+01

CAP-88 INFORMATION SHEET Date: Acril 1991 Source Category: Reactors ----Diablo Canyon Units 1 & 2 Facility: Diablo Canyon, CA Address: Population Assessment JCL File Name: DTARLO Array attached 35 12 39 120 51 7 Latitude: DIABLO Pop. File Name: Longitude: Individual Assessment JCL File Name: Distances (meters) 200 300 400 500 600 800 1200 1600 3200 4800 6400 8000 10000 15000 20000 30000 40000 50000 60000 80000 F1 F2 F3 Meat .008 .992 .000 Food Fractions: Urban Milk .000 1.00 .000 Rural Veg \_076 \_924 \_000 WEATHER DATA: Array attached STAR File Name: \_SBA0313 23190 Temperature: 19 °C WBAN: Header: 27.2 Lid Height: 600 meters SBA Code: Rainfall: 31 \_\_ cm/yr Set No: STACK Source: 1 2 3 5 4 6 Height (m): 69 Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec \_\_\_\_\_\_\_ MOMENTUM m/s \_\_\_\_\_ ENTERED Rise 0 0 0 0 0 0 meters Pasquill Category: A B C D E F G X ENTERED Rise Nuclide Class Diam Ci/yr

C-40

COMMENTS: Santa Barbars met data 5-yr 30 miles SSW of plant

File Name

DIABLO

### Distances

_	200	300	400_	500	600	800	1200	1600	3200	4800	6400
N											
NNW									.4		
NW										2_	8
WNW											
W											
WSW											
SW											
SSW											
S											
SSE											
SE											
ESE											10
E											
ENE											
NE_			_								
NNE											

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW	6								
NW									
WNW									
W									
WSW									
SW									
SSW									
S									
SSE									
SE									
ESE	40								
E	0								
ENE	6								
NE									
NNE									

INSTALLATION: DIABLO CANYON

LOCATION: 12 MI WSW OF SAN LUIS OBISPO

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: PACIFIC G&E

DOCKET NO.: 50-275 LICENSED PWR (MWT): 3.34E+03
THERMAL PWR (MWH): 1.66E+07 NET ELECTRIC PWR (MWH): 5.26E+06
COMMERCIAL OPERATION: 05/07/85 INITIAL CRITICALITY: 04/29/84

COOLING WATER SOURCE: PACIFIC OCEAN

UNIT NUMBER: 2 TYPE: PWR LICENSEE: PACIFIC G&E

DOCKET NO.: 50-323 LICENSED PWR (MWT): 3.41E+03
THERMAL PWR (MWH): 1.98E+07 NET ELECTRIC PWR (MWH): 6.23E+06
COMMERCIAL OPERATION: 03/13/86 INITIAL CRITICALITY: 08/19/85

COOLING WATER SOURCE: PACIFIC OCEAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	2.07E+00
CR-51	4.75E-06
MN-54	2.34E-04
CO-58	2.28E-04
CO-60	3.18E-05
KR-85	8.99E+00
KR-85M	1.03E-01
KR-87	1.58E-03
KR-88	1.16E-01
ZR-95	1.20E-05
I-131	6.99E-04
XE-131M	1.56E+01
I-133	3.61E-04
XE-133	2.88E+02
XE-133M	9.16E-01
CS-134	4.73E-05
XE-135	1.17E+01
XE-135M	4.45E-03
CS-137	2.98E-05
XE-138	9.76E-04

TOTAL AIRBORNE TRITIUM RELEASE 6.53E+01

- Data: April 1991 CAP-88 INFORMATION SHEET Source Category: Reactors
Facility: D.C. Cook Units 1 & 2  Address: Bridgman, M.1
Population Assessment JCL File Name: COOK
Latitude: 41 • 58 • 33 • COOK  Longitude: 86 • 33 • 59 • Pop. File Name:
Individual Assessment
Distances (meters)         200         300         400         500         600         800         1200         1600         3200         4800           6400         8000         10000         15000         20000         30000         40000         50000         60000         80000
Food Fractions: F1 F2 F3  Meat
Veg .076 .924 .000  WEATHER DATA:  Array attached STAR File Name:SBN0.257
WBAN: 14848 Temperature: 9 C Header: 0257 Lid Height: 950 meters Code: SBN Rainfall: 85 cm/yr Set No: 2
STACK Source: 1 2 3 4 5 6  Height (m):
AREA Source Diameter (m):
BUOYANT cal/sec
X ENTERED Rise 0 0 0 0 0 0 0 0 meters Pasquill Category: A B C D E F G
Nuclide Class Diam Ci/vr

COMMENTS: Benton Harbor DayTIME met data layr 10 miles N of plant.

Barry states above met data cannot be ured. He will use South Bend, IN 5-yr a ta (1967-1971)

File Name: COOK

Distances

	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											
NNW											
NW											
WNW											
W											
WSW											,
SW											
SSW							12		142	182	205
S							27		73	692	547
SSE								7	246	1088	302
SE							3_		126	141	172
ESE									214	231	240
E									197	186	199
ENE							-		284	143	297
NE							49		39	267	1321
NNE						12			30	66	196

-	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW									
NW									
WNW									
K									
WSW									
SW									
SSW	715	•							
5	1654	•							
SSE	1986								
SE	715								
ESE	1421								
E	820								
ENE	1159	•							
NE	2279								
NNE	758								

INSTALLATION: DONALD C. COOK

LOCATION: 11 MI SSW ST. JOSEPH, MI

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-315

THERMAL PWR (MWH): 2.42E+07

COMMERCIAL OPERATION: 08/27/75

LICENSEE: INDIANA MICHIGAN PWR
LICENSED PWR (MWT): 3.25E+03

NET ELECTRIC PWR (MWH): 7.47E+06

INITIAL CRITICALITY: 01/18/75

COOLING WATER SOURCE: LAKE MICHIGAN

UNIT NUMBER: 2 TYPE: PWR

DOCKET NO.: 50-316

THERMAL PWR (MWH): 7.41E+06

COMMERCIAL OPERATION: 07/01/78

LICENSEE: INDIANA MICHIGAN PWR
LICENSED PWR (MWT): 3.39E+03

NET ELECTRIC PWR (MWH): 2.32E+06

INITIAL CRITICALITY: 03/10/78

COOLING WATER SOURCE: LAKE MICHIGAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	3.90E-01
MN-54	3.39E-06
CO-58	6.83E-05
CO-60	3.42E-07
KR-85	1.52E+00
KR-85M	1.45E-01
KR-87	1.27E-01
KR-88	7.04E-01
NB-95	1.34E-07
ZR-95	3.33E-06
CD-109	3.49E-05
I-131	6.79E-03
XE-131M	8.08E-01
I-133	8.13E-04
XE-133	2.45E+02
XE-133M	1.25E+00
CS-134	3.40E-04
I-135	6.64E-05
XE-135	8.12E+00
XE-135M	1.35E-01
CS-137	8.04E-04
XE-138	1.04E-01
CE-139	4.92E-08

TOTAL AIRBORNE TRITIUM RELEASE 6.04E+00

## CAP-88 INFORMATION SHEET

Date: April 1991	Source Category:	Reactors
Facility: Fort Calhoun Address: Fort CAlhoun, NB		
Population Assessment JCL	File Name: FOI	RT
Latitude: 41 • 31 • 14 •	. File Name: FOI	RT
Individual Assessment JCL	File Name:	
Distances (meters) 200 300 400 500 600 6400, 8000 10000 15000 20000		1600 3200 4800 50000 60000 80000
Food Fractions: F1 F2 F3  Meat .008 .992 .00  Milk .000 1.00 .00  Veg .076 .924 .00	00 Rural	
WEATHER DATA:	TAR File Name: OMAO	991
WBAN: 14942 Temperate Header: 991 Lid Height Code: Rainfail: Set No:		
STACK Source: 1 2 Height (m):36 Diameter (m):	3 4	5 6
AREA Source Diameter (m ):		
BUOYANT cal/sec MOMENTUM m/s		
ENTERED Rise C C O Pasquill Category: A B C	0 0 0 D E F	G meters
Nuclide Class Diam Ci/yr:		

COMMENTS: Omaha/Eppley 10-yr met data 15 miles S of plant

## File Name: FORT

## Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N									2		
NNW										<b>6</b> 5	7
NW										81	375
WNW										202	6292
W									16	38	292
WSW								6	17	12	180
SW								13	12	15	10
SSW								12	13	8	12
							6		88	13	16
SSE								20	52	10	9
SE									<u>5</u> 1	8	14
ESE			,								
E											
ENE	·										
NE									6		24
NNE									_6	15	8

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW	23								
NW	23								
WNW	1378								
W	34								
WSW	49								
SW	32								
SSW	21								
S	108								
SSE	111								
SE	120								
ESE	6								
E	3								
ENE	3								
NE	8								
NNE	6								

INSTALLATION: FORT\_CALHOUN LOCATION: 19 MI N OMAHA, NE

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-285

THERMAL PWR (MWH): 8.34E+06

COMMERCIAL OPERATION: 06/20/74

LICENSEE: OMAHA PUBLIC PWR
LICENSED PWR (MWT): 1.50E+03

NET ELECTRIC PWR (MWH): 2.63E+06

INITIAL CRITICALITY: 08/06/73

COOLING WATER SOURCE: MISSOURI RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
ALPHA	4.08E-05
AR-41	1.73E+00
KR-85	6.07E+00
KR-85M	7.41E-03
SR-89	1.35E-07
SR-90	6.46E-08
I-131	3.07E-04
XE-131M	1.71E+01
I-133	2.29E-03
XE-133	7.55E+02
XE-133M	3.40E+00
XE-135	1.62E+00
CS-137	2.45E-06

TOTAL AIRBORNE TRITIUM RELEASE 3.85E+00

**CAP-88 INFORMATION SHEET** Source Category: Re ctors H.B. Robinson Unit 2 Facility: Hartsville, SC Address: Population Assessment JCL File Name: ROBINSON Array attached 34 • 29 • Latitude: 80 • 9 30 • Pop. File Name: ROBINSON Longitude: Individual Assessment JCL File Name: Distances (meters) <u>400</u> <u>500</u> <u>600</u> <u>800</u> <u>1200</u> <u>10000</u> <u>15000</u> <u>20000</u> <u>30000</u> <u>40000</u> 4800 1600 3200 300 8000 6400 50000 60000 80000 Food Fractions: F1 F2 Meat .442 .558 .000 Urban .399 .601 .000 .700 .300 .000 Milk Rural Veg WEATHER DATA: \_ Array attached STAR File Name: FL0600 Temperature: 19 13744 WBAN: Header: 600 Lid Height: \_\_800\_ meters Code: Ramail: 110 cm/yr FLO Set No: 1 42 STACK Source: 2 3 5 Height (m): Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec \_\_\_ MOMENTUM m/s X ENTERED Rise Pasquill Category: Nuclide Class Diam Ci/vr:

COMMENTS. Florence/Gilbert 5-yr met data 30 miles SE of plant

File Name: Robinson

_						ances					
	200	300	400	500	600	800	1200	1600	3200	4800	6400
K										9	
NNW									13	65	9
NW				1					· 35	60	39
WNW							117		13	4	
W						117			13	17	30
WSW						117			143	295	75
SW						74			222	153	40
SSW				96					199	35	16
S				28						71	50
SSE									13	317	514
SE			<sub>1</sub>					7	31	236	772
ESE									167	296	347
Е									79	304	590
ENE									40	203	210
NE									50	66	50
NNE				,					93	177	204

#### Distances 8000 10000 15000 20000 30000 40000 50000 60000 80000 N 87 NNW NW 39 WNW 34 W 21 WSW 45 SW 111 SSW 78 50 S SSE 474 1470 SE ESE 1990 1390 E ENE 197 NE 58 NNE 82

INSTALLATION: H.B.ROBINSON

LOCATION: 4.5 MI WNW HARTSVILLE, SC

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR LICENSEE: CAROLINA P&L

DOCKET NO.: 50-261 LICENSED PWR (MWT): 2.30E+03

THERMAL PWR (MWH): 1.06E+07 NET ELECTRIC PWR (MWH): 3.18E+06 COMMERCIAL OPERATION: 03/07/71 INITIAL CRITICALITY: 09/20/70

COOLING WATER SOURCE: ROBINSON IMPOUNDMENT

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	4.75E+00
CR-51	8.02E-07
MN-54	5.20E-08
CO-57	5.13E-09
CO-58	2.39E-06
CO-60	2.22E-05
KR-85	1.15E+02
KR-85M	4.65E+00
KR-87	1.56E-01
KR-88	2.76E-01
NB-95	4.63E-08
ZR-95	2.63E-08
I-131	1.07E-03
XE-131M	4.48E+00
I-133	6.82E-04
XE-133	8.38E+02
XE-133M	6.59E+00
CS-134	1.87E-08
XE-135	6.04E+01
XE-135M	7.77E-02
CS-137	2.06E-06

TOTAL AIRBORNE TRITIUM RELEASE 7.66E+00

N/A=NOT APPLICABLE

N/D=NOT DETECTED

N/R=NOT REPORTED

CAP-88 INFORMATION SHEET \_ Source Category: \_ Resctor-Date: \_Annal 1991 Facility: Haddam Neck Address: Hado, m Neck. CT HADDAM Population Assessment JCL File Name: Array attached Latitude: HADDAM Longitude: Pop. File Name: Individual Assessment JCL File Name: Distances (meters) 400\_\_\_500\_ 
 200
 300
 400
 500
 600
 800
 1200
 1600
 3200
 4800

 6400
 8000
 10000
 15000
 20000
 30000
 40000
 50000
 60000
 80000
 Food Fractions: F1 F2 F3 Meat \_008 \_992 \_000 Urban Milk .000 1.00 .000 Rural Veg .076 .924 .000 WEATHER DATA: STAR File Name: \_\_\_\_BDL1262 \_\_ Array attached 14740 Temperature: 10 ℃ WBAN: 1262 Header: Lid Height: \_700 meters BDL113\_\_cm/yr Code: Rainfall: Set No: \* 3 5 STACK Source: 1 2 4 Height (m). 41 Diameter (m): \_ AREA Source Diameter (m): MOMENTUM m/s X ENTERED Rise Pasquill Category: Nuclide Class Diam Ci/vr

COMMENTS: Hartford met deta 5-yr 25 miles NW of plant

File Name HADDAM

Distances

							•				
	200	300	400	500	600	800	1200	1600	3200	4800	6400
N							356		149	391	107
NNW								6	125	139	186
NW							3		120	120	44
WNW							3		23	208	828
W									438	368	377
WSW								65	131	66	239
SW	-							8	223	59	1434
SSW								135	29	112	157
s								106	33	1690	
SSE								_70	129	412	36
SE				_					201	558_	<u> </u>
ESE									48	184	224
Ė								9	67	186	81
ENE									323	346_	295
NE									192	632	584
NNE									290	163	185

	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	126								
NNW	209			,					4.1
NW	69								
WNW	692								
W	289								
WSW	115								
SW	252								
SSW									
S	44								
SSE	293								
SE	139								
ESE	51								
E	169								
ENE	265								
NE	198								
NNE	40								

INSTALLATION: HADDAM NECK

LOCATION: 9.5 MI SE MIDDLETOWN, CT

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-213

THERMAL PWR (MWH): 1.06E+07

COMMERCIAL OPERATION: 01/01/68

LICENSEE: CONN YANKEE ATOMIC PWR
LICENSED PWR (MWT): 1.82E+03

NET ELECTRIC PWR (MWH): 3.31E+06

INITIAL CRITICALITY: 07/24/67

COOLING WATER SOURCE: CONNECTICUT RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
ALPHA	1.86E-06
AR-41	7.14E-01
CO-60	3.54E-04
KR-85	1.94E+02
‴r -85M	3.38E+00
KR-87	3.07E+00
KR-88	4.55E+00
SR-90	8.70E-06
I-131	3.64E-02
XE-131M	4.05E+00
I-133	2.55E-03
XE-133	2.27E+03
XE-133M	7.69E+00
CS-134	6.11E-06
I-135	3.89E-02
XE-135	5.42E+01
XE-135M	5.45E-01
CS-137	1.50E-04
XE-137	3.65E-01
XE-138	1.36E+00

TOTAL AIRBORNE TRITIUM RELEASE 9.20E+01

Date:inri]		P-88 INFC	HMATION	Source	Category	Feact	tors	<del>-</del>
racuity	Inquen P		k 3			<del></del>		<b></b>
Address: _	suche nan	, NI			· · · · · · · · · · · · · · · · · · ·			_
	ion Assessn vrav attache		JCL File	Name:	_IN	DIAN		
	de: $\frac{41}{73}$		Pop. F	file Name	: IN	DIAN	<u></u>	_
Individu	al Assessm	ent	JCL File	Name:				_
Distances (me 200 300 6400 8000	400	<u>500</u> 0 <u>15000</u>	600 20000	800 30000	1200 40000	1600 50000	3200 60000	
Food Fraction	Meat. Milk	<u>.008</u> .9	92 <u>.000</u>		Urba Rura			
WEATHER DA					me:SW	F0185		
WBAN: Header: Code: Set No:	185	Lic	mperature I Height: infall:	12 900 112	meters cm/yr			
STACK Height Diamete		1.0	2	3	4	5	6	
AREA Diamet								
BUOYA	NT cal/sec NTUM m/s							
X ENTER	ED Rise I Category:	<u>A</u> <u>O</u>	) 0 B C	<u>o</u> -	0 0 E F	- <u>G</u> '	neters	
Nuclide !	Class Diam	Ci/yr:						
COMMENTS:	Newburg	h met a	.ta 5-yı	r 15 mi	les NN#	of mlar	nt	

File Name: INDIAN

## Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N							4=			158	280
NNW		_							30	280	305
NW										595	1155
WNW										98	
W								-	420	504	0
WSW									300	350	560
SW								3	10_	840	1505
SSW		, , ,					3		910_	263	1610
S							14		680_	53	<u>105</u>
SSE							210		1470	1068	595
SE						14		140	910	683	1190
ESE						7	546	62	630	298	_1085
E								280	630	263	980
ENE								18	1470	1890	<u> 1855</u>
NE									1050	2880	2135
NNE									1568	840	<u>875</u>

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	525								
NNW	1260								
NW	840								
WNW	875								
W	630								
WSW	560								
SW	945								
SSW	2415								
S	245								
SSE	385								
SE	1190								
ESE	<b>3</b> 85								
E	875								
ENE	2100	-							
NE	2660								
NNE	2240								

INSTALLATION: INDIAN POINT 1-2 LOCATION: 3 MI SW PEEKSKILL, NY

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: CONSOLIDATED EDISON DOCKET NO.: 50-3 LICENSED PWR (MWT): 6.15E+02

THERMAL PWR (MWH): 0.00E+00 NET ELECTRIC PWR (MWH): 0.00E+00

COMMERCIAL OPERATION: INITIAL CRITICALITY: 08/02/62

COOLING WATER SOURCE: HUDSON RIVER

UNIT NUMBER: 2 TYPE: PWR LICENSEE: CONSOLIDATED EDISON
DOCKET NO.: 50-247 LICENSED PWR (MWT): 2.76E+03
THERMAL DWR (MWH): 1 05E+07

THERMAL PWR (MWH): 1.95E+07 NET ELECTRIC PWR (MWH): 6.06E+06 COMMERCIAL OPERATION: 08/01/74 INITIAL CRITICALITY: 05/22/73

COOLING WATER SOURCE: HUDSON RIVER

#### AIRBORNE EFFLUENTS ACTIVITY (CI) NUCLIDES RELEASED 7.40E-01 AR-41 MN-54 1.56E-03 FE-55 3.14E-06 CO-58 1.52E-04 CO-60 5.65E-03 NI-63 6.14E-07 KR-85M 2.16E-01 KR-87 4.57E-02 KR-88 2.85E-01 SR-90 1.09E-08 I - 1314.02E-05 I - 1332.97E-03 XE-133 2.05E+02 XE-133M 4.34E-01 XE-135 1.13E+01 XE-135M 4.99E-01 CS-137 1.77E-03

TOTAL AIRBORNE TRITIUM RELEASE 1.69E+00

N/A=NOT APPLICABLE N/D=NOT DETECTED N/R=NOT REPORTED

XE-138

3.52E-03

INSTALLATION: INDIAN POINT 3 LOCATION: 3 MI SW PEEKSKILL, NY

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 3 TYPE: PWR LICENSEE: PWR AUTH - STATE OF NY DOCKET NO.: 50-286 LICENSED PWR (MWT): 2.76E+03
THERMAL PWR (MWH): 2.14E+07 NET ELECTRIC PWR (MWH): 6.71E+06
COMMERCIAL OPERATION: 08/30/76 INITIAL CRITICALITY: 04/06/76
COOLING WATER SOURCE: HUDSON RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	3.58E-01
CO-58	8.69E-06
KR-85	3.26E+00
KR-85M	1.46E-01
<del>87</del>	5.92E-02
KR-88	2.08E-01
I-131	3.41E-03
XE-131M	8.32E+00
I-133	5.47E-04
XE-133	2.87E+02
XE-133M	2.02E+00
CS-134	1.49E-06
XE-135	8.24E+00
XE-135M	1.32E-02
CS-137	2.40E-06
XE-138	3.30E-03

TOTAL AIRBORNE TRITIUM RELEASE 4.56E+00

CAP-88 INFORMATION SHEET Source Category: Reactors Dail: 1001 Joseph M. Farley Units 1 & 2 Facility: Dothan, AL Address: Population Assessment JCL File Name: FERLEY Array attached Latitude: FARLEY Pop. File Name: Longitude: Individual Assessment JCL File Name: Distances (meters) 3200 4800 1600 300 1200 600 800 200 400 15000 20000 8000 10000 30000 6400 40000 00008 50000 600C0 Food Fractions: F1 F2 F3 .008 .992 .000 .000 1.00 .000 Urban Meat Milk Rural .076 .924 .000 Vea WEATHER DATA: STAR File Name: \_\_TLH0663 Array attached Temperature: 20 Lid Height: 850 93805 WBAN: C 663 Header: Lid Height: meters TLHRainfall: Code: 164 cm/yr Set No: STACK Source: 2 3 5 1 6 44 Height (m): Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec MOMENTUM m/s ENTERED Rise Pasquill Category: **Nuclide** Class Diam Ci/vr: COMMENTS: Tallahassee, FL 5-yr met data 75 miles SW of plant

## File Name: EARLEY\_\_\_

## Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N										6	28
NNW										32	37
NW									5	16	32
MUM										11	42
W								17	17	11	52
WSW								5			79 •
SW									17	21	32
SSW										11	74
S											21
SSE											
SE										43	74
ESE											86
E								~			31
ENE										23	
NE										6_	12
NNE										44	49

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	971								
NNW	79								
NW	121								
WNW	47								
W	47								
WSW	52								
SW	37								
SSW	74								
S	129								
SSE	37								
SE	129								
ESE									
E									
ENE	37								
NE	55								
NNE	147								

THEMSTISMICH TOCHNUM PARTON 1

INSTALLATION: JOSEPH M. FARLEY 1

LOCATION: DOTHAN, AL

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: ALABAMA PWR

DOCKET NO.: 50-348 LICENSED PWR (MWT): 2.65E+03
THERMAL PWR (MWH): 1.92E+07 NET ELECTRIC PWR (MWH): 5.91E+06
COMMERCIAL OPERATION: 12/01/77 INITIAL CRITICALITY: 08/09/77

COOLING WATER SOURCE: CHATAHOOCHEE RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	2.63E+01
CR-51	2.40E-04
MN-54	2.85E-05
CO-58	7.47E-05
FE-59	1.45E-05
CO-60	4.06E-05
ZN-65	1.30E-05
KR-85	6.86E+02
KR-85M	6.11E-01
KR-87	6.27E-02
KR-88	2.02E-01
I-131	1.18E-03
XE-131M	1.15E+00
I-133	1.30E-06
XE-133	1.83E+02
XE-133M	2.13E+ <b>00</b>
CS-134	1.48E-08
XE-135	6.03E+01
XE-135M	2.58E-07
CS-137 '	1.28E-06
XE-138	8.21E-02
CE-141	3.00E-07
CE-144	1.45E-07

### TOTAL AIRBORNE TRITIUM RELEASE 1.16E+02

N/A=NOT APPLICABLE

N/D=NOT DETECTED

N/R=NOT REPORTED

INSTALLATION: JOSEPH M. FARLEY 2

LOCATION: DOTHAN, AL

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR LICENSEE: ALABAMA PWR

DOCKET NO.: 50-364

THERMAL PWR (MWH): 2.29E+07

COMMERCIAL OPERATION: 07/30/81

LICENSED PWR (MWT): 2.65E+03

NET ELECTRIC PWR (MWH): 7.17E+06

INITIAL CRITICALITY: 05/05/81

COOLING WATER SOURCE: CHATAHOOCHEE RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	3.56E+01
KR-85	4.58E+02
I-131	2.36E-06
I-133	5.00E-09
133	7.41E+01
XE-133M	1.90E-01
CS-134	9.36E-09
XE-135	2.38E+01
CS-137	1.46E-07
XE-138	5.32E-02

TOTAL AIRBORNE TRITIUM RELEASE 5.96E+01

### CAP-88 INFORMATION SHEET

Date:Ap	r11 1991			Source	Categor	Reactor	'S	
Facility: Address:	Kewaunee Carlton,	WI						
Рорі	ulation Assessi		JCL File	Name:	KE	VAUNEB		
_		20 • 35	-	ile Name	E KE	WAUNEE		
a low	10081 A3563311	iei if	JUL PILE	Name.				
	3000 400	500 15000	_600 20000	800 30000	1200 40000	1600 33 50000 600	200 <u>480</u>	
Food Fract	ions: Meat Milk Veg	F1 F2 .008 .9 .000 1.0 .076 .92	00 -000	)	Urb Rura			
WEATHER		ed	STA	R File N	ame: <u>ER</u>	B0776		
WBAN: Header: Code: Set No:	74898 776 SRB	Lid	nperature Height: nfall:	8 850 79	_ °C _ meters _ cm/yr			
Heig	CK Source: pht (m): neter (m):	1	2	3	4	5	6	
	A Source neter (m³):							
BUC	YANT cal/sec MENTUM m/s							
X ENT	ERED Rise quill Category:	0 B	<u>C</u>	<u>0</u> -	0 0 E F	0 met	ers	
Nuclide	Class Diam	Ci/vr·						

COMMENTS: Green Bay 10-yr met data ~25 WNE of plant

File Name: KEWAUNEE

Distances

							-				
_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N					_				16	31	53
NNW			_					8	20	26	47
NW									11	47	84
WNW									16	58	79
W								4	27	26	53
WSW									37	84	42
SW		-						4	16	21	53
SSW								4	16	79	126
S										5	26
SSE											
SE											
ESE											
E											
ENE											
NE											
NNE									16	26_	42

#### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	79			-					
NNW	79								
NW	74								
MNM	63								
W	68								
WSW	500	-							
SW	79								
SSW	94		·						
S	47								
SSE									
SE			-						
ESE									
E						-			
ENE									
NE									
NNE	21								

INSTALLATION: KEWAUNEE

LOCATION: 27 MI ESE GREEN BAY, WI

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-305

THERMAL PWR (MWH): 1.22E+07

COMMERCIAL OPERATION: 06/16/74

LICENSEE: WISCONSIN PUB. SER.

LICENSED PWR (MWT): 1.65E+03

NET ELECTRIC PWR (MWH): 3.91E+06

INITIAL CRITICALITY: 03/07/74

COOLING WATER SOURCE: LAKE MICHIGAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	7.57E-02
FE-55	<4.24E-05
CO-58	1.59E-05
CO-60	1.27E-05
AS-76	4.08E-08
KR-85	1.88E-01
KR-85M	1.80E-03
SR-89	<4.83E-05
SR-90	<9.25E-06
MO-TC-99M	6.50E-08
RU-103	2.01E-08
TC-104	1.02E-06
I-131	3.45E-04
XE-131M	1.47E-03
I-132	5.29E-03
I-133	2.51E-05
XE-133	6.42E+00
XE-133M	1.43E-02
XE-135	4.97E-02
CS-137	5.39E-07
UNIDENTIFIED	<2.23E+01

TOTAL AIRBORNE TRITIUM RELEASE <7.15E+00

CAP-88 INFORMATION SHEET Source Category: Reactors Date: \_ Arril 1991 M ine Yankee Facility: Wisca-37t, ME Address: Population Assessment MAINE JCL File Name: Array attached 2 • Latitude: Longitude: 69 41 45 Pop. File Name: MAINE Individual Assessment JCL File Name: Distances (meters) 200 300 400 500 600 800 1200 1600 3200 4800 6400 8000 10 00 15000 20000 30000 40000 50000 60000 80000 Food Fractions: F1 F2 F3 Meat \_008 \_992 \_000 Urban \_000 1\_00 \_000 Rural Veg \_076 \_924 \_000 WEATHER DATA: STAR File Name: NHZ0180 Array attached Temperature: 7 14611 WBAN: Header: 180 Lid Height: 600 meters Code: 111 Rainfall: cm/yr Set No: 5 STACK Source: 1 2 3 4 6 Height (m): 53 Diameter (m): AREA Source Diameter (m<sup>3</sup>): BUOYANT cal/sec \_ \_\_\_ MOMENTUM m/s O O O O O O Meters

B C D E F G X ENTERED RISA Pasquill Category: Nuclide Class Diam Ci/yr:

COMMENTS: Brunswick lo-yr met data 15 miles SW of plant

### THE Name: MAINE

					Dist	ances					
	200	300	400	500	600	800	1200	1600	3200	4800	6400
N		0.000						18	43	108	131
NNW							3		58	43	
NW				0			•	6	104	52	9
WNW						٥			48	48	37
W									49	76	104
WSW							0		23	52	43
SW							•	3		39	3
SSW									22	13	40
S								9	16	16	125
SSE							9		24	23	63
SE						5			10	37	30
ESE								15	15	12	71
Ε							11		37	20	129
ENE							9		13	48	118
NE			•					3	2	18	420
NNE									43	86	517

8000 10000 15	5000 20000	30000 40000	SOORO	COOOO	
			30000	00000	80000
N 49			<del> </del>	•	
NNW					
NW 57					
WNW 104					
W 261					
WSW 37					
SW 6					
SSW 66					
S 267					
SSE 57					
SE 106					
ESE 37					
E 24					
ENE 27					
NE 617					
NNE 260					

INSTALLATION: MATNE YANKEE LOCATION: 3.9 MI S WICASSETT, ME

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-309

THERMAL PWR (MWH): 1.67E+07

COMMERCIAL OPERATION: 12/28/72

LICENSEE: MAINE YANKEE ATOMIC PWR
LICENSED PWR (MWT): 2.63E+03

NET ELECTRIC PWR (MWH): 5.02E+06

INITIAL CRITICALITY: 10/23/72

COOLING WATER SOURCE: BACK RIVER

NUCLIDES RELEASED ACTIVITY (CI	)
	-
CO-58 2.76E-06	
CO-60 1.85E-05	
KR-85 1.56E+00	
KR-85M 3.52E-03	
KR-88 7.00E-05	
CD-109 1.70E-06	
I-131 4.44E-04	
XE-131M 9.04E-02	
I-133 1.08E-04	
XE-133 6.71E+01	
XE-133M 4.94E-01	
XE-135 2.73E+00	
CS-137 2.80E-07	

TOTAL AIRBORNE TRITIUM RELEASE 6.42E+00

CAP-88 INFORMATION SHEET
1991 Source

Date: Apr	17 7227			200108	Category.	Reactor	<u> </u>	-
Facility: Address:	McGuire 1 Cowans Fo		NC					
	ation Assessm Arrav attache		JCL File	Name:	MCG	UIRE		
Latitud Longit		25 55 55 56 53		īle Name	: MCG	UIRE	<del></del>	
Individ	lual Assessme	nt	JCL File	Name:				•
Distances (m 200 3 6400 80	00 400	500 15000	600 20000	800 30000	1200	1600 320 50000 600	0 00 E	4800 0000
Food Fractio		F1 F2	<b>F3</b>		Urba	n		
	Milk	.000 1.0 .076 .9	000-00	0	Rura	ı		
WEATHER D	<u>DATA:</u> Array attache	d	STA	R File Na	me. CLT	0682		
WBAN: Header: Code: Set No:	13881 682	Lid	nperature Height: *fall:	13. 1000 110	meters cm/yr			
Heigh	K Source: t (m): eter (m):	1 40		3	4	5	6	<u>-</u>
	Source eter (m²):							_
BUOY	'ANT cal/sec ENTUM m/s				-			<u>-</u>
_X_ ENTE	RED Rise iill Category:	0 0 A B	_ <u>C</u>	<u>0</u> <u>0</u>	0 E F	G met	ers	
Nuclide	Class Diam	Ci/yr·						
								<del>-</del>
								<del>-</del>

COMMENTS: Charlotte 5-yr met data 20 miles SSE of plant

File Name: MCGUIRE

- - Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											29
NNW									32.	41	57
NW									22	67	137
WNW									3	115	57
W								6	115	19	38
WSW									29	48	115
SW									10	0	3
SSW										19	16
S									13		45
SSE									16	26	29
SE									22	64	93
ESE								19	80	38	54_
E										128	29
ENE									0	67	153
NE										41	32
NNE										38	10

Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	26								
NNW	191								
NW	73								
WNW	26								
W									
WSW	166								
SW	309								
SSW	10								
S	41								
SSE	89								
SE	115								
ESE	35								
Е	93								
ENE	57							<u>.</u>	
NE	160								
NNE	99								

LOCATION: 17 MI N OF CHARLOTTE, NC

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: DUKE PWR

DOCKET NO.: 50-369

THERMAL PWR (MWH): 2.26E+07

COMMERCIAL OPERATION: 12/01/81

LICENSED PWR (MWT): 3.41E+03

NET ELECTRIC PWR (MWH): 7.39E+06

INITIAL CRITICALITY: 08/08/81

COOLING WATER SOURCE: LAKE NORMAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVIT. (CI)
NA-24	1.60E-09
CL-38	6.07E-08
K-40	2.64E-05
AR-41	8.77E+00
CR-51	1.88E-06
MN-54	8.25E-07
CO-57	1.93E-06
CO-58	6.48E-05
CO-60	7.28E-05
BR-82	9.58E-08
KR-85	3.69E+01
KR-85M	4.92E+00
KR-87	7.05E-01
KR-88	4.64E+00
RB-88	1.70E-04
\$R-89	1.03E-06
SR-92	4.63E-08
NB-95	5.58E-10
TC-99M	2.31E-08
RU-103	1.20E-07
I-131	5.58E-03
XE-131M	1.07E+01
I-132	1.20E-03
I-133	1.83E-03
XE-133	1.79E+03
XE-133M	2.44E+01
CS-134	4.73E-05
I-134	1.12E-07
I-135	6.91E-07
XE-135	7.09E+01
XE-135M	1.39E-02
CS-136	8.52E-06
CS-137	1.11E-04
CS-138	6.41E-07
AE-130	1.08E-03
BA-139	4.04E-07
CE-141	2.55E-10
PB-212	3.64E-07
BI-214	4.13E-05
PB-214	1.27E-05

TOTAL AIRBORNE TRITIUM RELEASE 2.40E+01

LOCATION: 17 MI N OF CHARLOTTE, NC

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR LICENSEE: DUKE PWR

DOCKET NO.: 50-370 LICENSED PWR (MWT): 3.41E+03
THERMAL PWR (MWH): 2.41E+07 NET ELECTRIC PWR (MWH): 8.05E+06
COMMERCIAL OPERATION: 03/01/84 INITIAL CRITICALITY: 05/08/83

COOLING WATER SOURCE: LAKE NORMAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
NA-24	1.60E-09
CL-38	6.07E-08
K-40	2.64E-05
AR-41	8.77E+00
CR-51	1.88E-06
MN-54	8.25E-07
CO-57	1.93E-06
CO-58	6.48E-05
CO-60	7.28E-05
BR-82	9.58E-08
KR-85	3.69E+01
KR-85M	4.92E+00
KR-87	7.05E-01
KR-88	4.64E+00
RB-88	1.70E-04
SR-89	1.03E-06
SR-92	4.63E-08
NB-95	5.58E-10
TC-99M	2.31E-08
RU-103	1.20E-07
I-131	5.58E-03
XE-131M	1.07E+01
I-132	1.20E-03
I-133	1.83E-03
XE-133	1.79E+03
XE-133M	2.44E+01
CS-134	4.73E- <b>0</b> 5
I-134	1.12E-07
I-135	6.91E-07
XE-135	7.09E+01
XE-135M	1.39E-02
CS-136	8.52E-06
CS-137	1.11E-04
CS-138	6.41E-07
XE-138	1.08E-03
BA-139	4.04E-07
CE-141	2.55E-10
PB-212	3.64E-07
BI-214	4.13E-05
PB-214	1.27E-05

TOTAL AIRBORNE TRITIUM RELEASE 2.40E+01

Date: April 19	CAP-88 INFOI	RMATION S S	HEET ource Categ	ory: Reac	tors
FACULY	lstone Units 1 erford, CT	, 2 & 3			
	Assessment attached	JCL File N	ame:	MILLSTON	
Latitude:	41 • 18 ; 30 · 5 ·	Pop. File	Name:	MILLSTON	
Individual As	ssessment	JCL File N	ame:		
Distances (meters) 300 6400. 8000			300 <sub>120</sub>		3200 4800 60000 80000
Food Fractions:	Meat .008 .99 Milk .000 1.0 Veg .076 .9	2 .000 00 .000		Jrban Rural	
WEATHER DATA: Array	attached		File Name:	BDRO558	
	58 Lid	nperature: Height: nfall:	11 C 700 mete 98 cm/	-	
STACK Sou Height (m): Diameter (n	110	2 	3	4 :	5 6
AREA Sou Diameter (n	•				
	cal/sec M m/s				
X ENTERED I	Rise $0$ $0$ tegory: A B	- <u>°</u> -	0 0 D E	0 0 F G	meters
Nuclide Class	S Diam Ci/yr:				

COMMENTS: Bridgeport met data (both near Long Island Sound 60 miles WSW of plant 5- yr

#### File Name : MILLSTON

h	4	•	+	•	_	_	04	
	1	ч	т	71	מו	Г	р,	

	200	300	400	500	600	800	1200	1600	3200	4800	6400
N								174	1500	771	770
NNW						357			876	541	541
NW									1124	527	527
WNW									1712	834	834
W									958	727	727
WSW									1560	127	128
SW									33	53	52
SSW											
S											
SSE											
SE											
ESE									146		
E						421			1029	649	649
ENE								306	590	1738	1737
NE						375			1436	450	449
NNE								14	1363	1125	1125

#### Distances

8000	10000	15000	20000	30000	40000	50000	60000	80000
156	5							
970	)							
561								
119	)							
542	2							
998	}							
1076	ŝ							
4338	3							
1814	7							
70	1							
	156 970 561 119 542 998 1076 4338 1814	1076 4338 18147 701	156 970 561 119 542 998 1076 4338 18147	156 970 561 119 542 998 1076 4338 18147	156 970 561 119 542 998 1076 4338 18147	156 970 561 119 542 998 1076 4338 18147	156 970 561 119 542 998 1076 4338 18147	156 970 561 119 542 998 1076 4338 18147

INSTALLATION: MILLSTONE 1

LOCATION: 3.2 MI WSW NEW LONDON, CT

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: NORTHEAST NUC.ENERGY DOCKET NO.: 50-245 LICENSED PWR (MWT): 2.01E+03 THERMAL PWR (MWH): 1.70E+07 NET ELECTRIC PWR (MWH): 5.54E+06 COMMERCIAL OPERATION: 03/01/71 INITIAL CRITICALITY: 10/26/70

COOLING WATER SOURCE: LONG ISLAND SOUND

AIRBORNE EFFLUENTS	,
NUCLIDES RELEASED	ACTIVITY (CI)
CR-51	7.28E-04
MN-54	6.01E-05
CO-58	4.03E-05
FE-59	6.00E-06
CO-60	8.83E-04
ZN-65	1.62E-03
KR-85	1.99E+02
KR-85M	4.82E+00
KR-87	3.15E+ <b>0</b> 1
KR-88	2.51E+00
SR-89	5.79E-04
SR-90	3.67E-06
I-131	2.97E-03
I-133	1.45E-02
XE-133	4.05E+02
XE-135	5.32E+01
XE-135M	3.60E+01
CS-137	1.01E-04
XE-138	1.46E+02
BA-140	5.62E-04
LA-140	1.73E-04

TOTAL AIRBORNE TRITIUM RELEASE 7.21E+01

N/A=NOT APPLICABLE N/D=NOT DETECTED

N/R=NOT REPORTED

INSTALLATION: MILLSTONE 2

LOCATION: 3.2 MI WSW NEW LONDON, CT

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR LICENSEE: NORTHEAST NUC. ENERGY DOCKET NO.: 50-336 LICENSED PWR (MWT): 2.70E+03 THERMAL PWR (MWH): 1.81E+07 NET ELECTRIC PWR (MWH): 5.73E+06 COMMERCIAL OPERATION: 12/26/75 INITIAL CRITICALITY: 10/17/75

COOLING WATER SOURCE: LONG ISLAND SOUND

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
ALPHA	<5.56E-07
CR-51	5.98E-06
MN-54	6.34E-06
CO~58	3.73E-04
33 -60	4.61E-05
KR-85	4.25E+00
KR-85M	1.56E-05
SR-89	<8.22E-07
SR-90	<4.82E-08
I-131	5.08E-02
XE-131M	2.45E+00
I-133	2.61E-02
XE-133	7.32E+02
XE-133M	1.71E+00
CS-134	1.29E-05
XE-135	1.29E+02
CS-137	1.62E-05

TOTAL AIRBORNE TRITIUM RELEASE 1.04E+02

INSTALLATION: MILLSTONE 3

LOCATION: 3.2 MI WSW OF NEW LONDON, CT

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 3 TYPE: PWR

DOCKET NO.: 50-423

THERMAL PWR (MWH): 2.33E+07

COMMERCIAL OPERATION: 04/23/86

LICENSEE: NORTHEAST NUC. ENERGY

LICENSED PWR (MWT): 3.41E+03

NET ELECTRIC PWR (MWH): 7.67E+06

INITIAL CRITICALITY: 01/23/86

COOLING WATER SOURCE: LONG ISLAND SOUND

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
ALPHA	<8.73E-06
MN-54	5.27E-07
CO-58	3.11E-04
CO-60	1.56E-05
KR-85M	2.14E-03
Y-88	6.31E-07
SR-89	<6.96E-06
SR-90	<4.21E-06
I-131	9.45E-03
XE-131M	2.03E-01
I-133	4.45E-03
XE-133	7.77E+01
XE-133M	5.02E-01
XE-135	5.96E+00
CS-137	1.63E-05
ND-147	7.94E-05

TOTAL AIRBORNE TRITIUM RELEASE 7.16E+01

CAP-88 INFORMATION SHEET

Date: April 1991	Source Category: Reactors
Facility: North Anna Units 1&2 Address: Mineral, VA	
Population Assessment JCL File	ie Name: NORTHANN
Latitude: 38 • 3 • 38 • 1 • 26 • Pop. I	File Name: NORTHANN
Individual Assessment JCL File	le Name:
Distances (meters) 200 300 400 500 600 6400 8000 10000 15000 20000	800 1200 1600 3200 480 30000 40000 50000 60000 8000
Food Fractions: F1 F2 F3  Meat .442 .558 .000  Milk .399 .608 .000  Veg .700 .300 .000	Urban Rural
WEATHER DATA: Array attached STA	
	e: 14 °C _800 _ meters _112 _ cm/yr
STACK Source: 1 2 Height (m): 48 Diameter (m):	3 4 5 6
AREA Source Diameter (m²):	
BUOYANT cal/sec MOMENTUM m/s	
X ENTERED Rise 0 0 0 Pasquill Category: A B C	
Nuclide Class Diam Ci/yr:	
COMMENTS: Gordonsville 5-yr me	t data 15 miles Nw of tlant

File Name: NORTHANN

			• • •		Dist	ances					
	200	300	400	500	600	800	1200	1600	3200	4800	640C
N_								27		128	85
NNW								6		32	21
NW									. 9	26	
WNW									5	31	31
W									9	9	9
WSW									40	9	14
SW									17	22	52
SSW									9		35
S									9	22	9
SSE									9	14	35
SE									14	9	5
ESE									11		11
Ε									3	27	27
ENE									3	102	17
NE								3		38	59
NNE								6		53	181

			Distances							
	8000	10000	15000	20000	30000	40000	50000	60000	80000	
N	123									
MNM	17									
NW	11									
WNW	14									
W	22									
WSW	52									
SW	40									
SSW	62									
S	31									
SSE	5									
SE	14									
ESE	81									
Ε	6									
ENE	43									
NE	17									
NNE	102									

INSTALLATION: NORTH ANNA

LOCATION: 40 MI NW RICHMOND, VA

#### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

ACTIVITY (CI)

8.56E-04

2.07E-06

7.38E-06

7.61E-06

7.04E-05

3.64E+00

2.96E-02

3.71E-04

4.80E-04

4.07E-06

5.75E-10

1.44E-07

1.92E-05

1.56E-03

5.33E-09

1.95E+00 6.65E-06

1.21E-02

4.74E+02

5.83E-01

1.54E-04

1.20E-05 2.60E+00

4.94E-04

9.19E-05

1.32E-06

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-338

DOCKET NO.: 50-338
THERMAL PWR (MWH): 2.20E+07

COOLING WATER SOURCE: LAKE ANNA

UNIT NUMBER: 2 TYPE: PWR

DOCKET NO.: 50-339

COOLING WATER SOURCE: LAKE ANNA

LICENSEE: VIRGINIA ELECTRIC & PWR LICENSED PWR (MWT): 2.89E+03 THERMAL PWR (MWH): 2.20E+07 NET ELECTRIC PWR (MWH): 6.90E+06 COMMERCIAL OPERATION: 06/06/78 INITIAL CRITICALITY: 04/05/78

LICENSEE: VIRGINIA ELECTRIC &PWR LICENSED PWR (MWT): 2.89E+03 THERMAL PWR (MWH): 2.50L-07 NET ELECTRIC PWR (MWH): 7.88E+06 COMMERCIAL OPERATION: 12/14/80 INITIAL CRITICALITY: 06/12/80

AIRTONE EFFLUENTS

NUCLIDES RELEASED AR-41

MN-54

FE-55

CO-58

CO-60

KR-85 KR-85M

KR-87

KR-88

RB-88

NB-95 AG-110M

SB-122

I - 131

TE-131M

XE-131M

I-132

I-133

XE-133

XE-133M

CS-134

I-135

XE-135

CS-137

CS-138

CE-143

TOTAL AIRBORNE TRITIUM RELEASE 9.44E+01

N/A=NOT APPLICABLE

N/D=NOT DETECTED

N/R=NOT REPORTED

**CAP-88 INFORMATION SHEET** Source Category: Reactors-Date: - April 1991 Oconee 1, 2 & 3 Facility: Seneca, SC Address: OCONEE Population Assessment JCL File Name: Array attached 34 ° 47 ° 30 ° Latitude: OCONEE 82 • 53 • 54 • Longitude: Pop. File Name: Individual Assessment JCL File Name: Distances (meters) 4800 1600 3200 400 800 1200 200 300 500 600 10000 15000 20000 30000 40000 50000 6400, 8000 60000 80000 Food Fractions: F1 F2 Meat .008 .992 .000 Urban .000 1.00 .000 Milk Rural .076 .924 .000 Vea WEATHER DATA: Array attached STAR File Name: GSP0528 13 03870 WBAN: Temperature: \_\_\_ 528 1000 meters Header: Lid Height: 125 cm/yr Code: Rainfall: Set No: STACK Source: 1 3 2 4 5 Height (m): 61 Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec MOMENTUM m/s \_ ENTERED Rise Pasquill Category:

Nuclide

Class Diam Ci/vr:

COMMENTS: Greenville/Sparta 5-yr met data ~35 miles NE of plant

### Tile Name: OCONEE

#### Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N										35	.123
NNW									29.7	374	. 884
NW									311	515	465
WNW									26	53	65
W									21	90	84
WSW									12	79	30
SW									31	99	37
SSW									48	137	12
s									10	12	36
SSE									12	18	26
SE									10	27	22
ESE									18	34	14
E									22	44	68
ENE									18	38	81_
NE									15	33	<u>76</u>
NNE									35	215	46

#### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
ĸ	27						<b>.</b>		
NNW	44								
NW	78								
WNW	58								
W	- 81				,				
WSW	79		_						
SW	28								
SSW	6								
S	203								
SSE	133								
SE	66								
ESE	97			•					
E	308								
ENE	142								
NE	89								
NNE	8_								

#### -INSTALLATION:\_OCONEE

LOCATION: 30 MI W GREENVILLE, SC

#### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: DUKE PWR

**DOCKET NO.: 50-269** LICENSED PWR (MWT): 2.57E+03 THERMAL PWR (MWH): 2.21E+07

COMMERCIAL OPERATION: 07/15/73

EICENSED FWR (MWH): 2.37E+03

NET ELECTRIC PWR (MWH): 7.19E+06

INITIAL CRITICALITY: 04/19/73

COOLING WATER SOURCE: KEOWEE LAKE

UNIT NUMBER: 2 TYPE: PWR LICENSEE: DUKE PWR

DOCKET NO.: 50-270 LICENSED PWR (MWT): 2.57E+03

THERMAL PWR (MWH): 1.72E+07

COMMERCIAL OPERATION: 09/09/74

LICENSED PWR (MWI): 2.37E+03

NET ELECTRIC PWR (MWH): 5.54E+06

INITIAL CRITICALITY: 11/11/73

COOLING WATER SOURCE: KEOWEE LAKE

UNIT NUMBER: 3 TYPE: PWR LICENSEE: DUKE PWR

DOCKET NO.: 50-287 LICENSED PWR (MWT): 2.57E+03 THERMAL PWR (MWH): 1.80E+07

COMMERCIAL OPERATION: 12/16/74

INITIAL CRITICALITY: 09/05/74

ACTIVITY (CI)

COOLING WATER SOURCE: KEOWEE LAKE

AIRBORNE	EFFLUENTS
NUCLIDES	RELEASED

KR-87

NA-24 1.05E-07 AR-41 5.12E+00 CR-51 4.28E-05 MN-54 8.00E-06 CO-57 2.87E-07 CO-58 1.19E-04 CO-60 2.06E-04 KR-85 1.69E+03 KR-85M 9.65E+00

1.53E+00 KR-88 8.79E+00 5.67E-02 RB-88 RB-89 1.35E-05

SR-89 1.50E-02 SR-90 3.03E-07

SR-92 1.33E-06

NB-95 2.49E-06 ZR-95 9.26E-07

NB-97 3.91E-06

TC-99M 2.45E-06

RU-103 3.33E-05 RU-106 3.48E-05

AG-110M 8.48E-06

SB-122 3.51E-07

SB-125 4.67E-06 I - 1318.29E-02

XE-131M 3.72E+02

I - 1321.46E-03

#### INSTALLATION: OCONEE\_\_\_\_

I-133	1.91E-02
XE-133	2.34E+04
XE-133M	1.87E+02
CS-134	5.90E-04
I-134	1.87E-04
1-135	4.70E-03
XE-135	1.87E+02
XE-135M	1.26E+00
CS-136	6.82E-06
CS-137	2.10E-03
CS-138	4.82E-03
LA-140	1.33E-08
CE-141	2.02E-07
CE-143	3.78E-05
CE-144	6.84E-06

#### TOTAL AIRBORNE TRITIUM RELEASE 4.60E+01

Date:	April 199	CAP-	BB INFOR	MATION 	SHEET Source	Category:	ne c	tors	
Facility	/: <u> </u>	Palisade	<b>)</b>						<u>.</u> .
Addres		South Ha	ven. M	I					_
	Population A Array Latitude: Longitude:	attached				<u> </u>	ALISADI		<b>-</b>
	Longitude:	<u>86</u>	18 55	Pop. Fi	e Name:		ALISADI	<u> </u>	_
	Individual As	sessment	t	JCL File	Name:			<del></del>	-
	(meters) 300 8000		500 15000	600 20000	800 30000	1200 40000	1600 50000	3200 60000	4800 80000
	Fractions:	Milk _		F3 000 00000 4000		Urba Rural			
WEAT	HER DATA: Array	attached		STAF	File Na	me: <u>MK</u> (	30251	<del></del>	_
WBAN Heade Code: Set No	er: 2 <u>F</u>	40 51 KG		perature: Height: fall:	. 950	T meters cm/yr			
	STACK Sour Height (m): Diameter (m		1 58.1	2	3	4	5		-
	AREA Sour Diameter (m						<del>-</del>		
	BUOYANT O	cal/sec _ M m/s _	<del></del> .						<del></del>
_X_	ENTERED R Pasquill Cate	ise _ egory:	0 0 A B	<u> </u>	<u>O</u> _	0 0 E F	<u> </u>	meters	
Nuclid	de Class	Diam C	i/yr:						

COMMENTS: Wuskegon Co met a ta 5-yr 50 miles N of plant

File Name: PALISADS

	Distances										
	200	300	400	500	600	800	1200	1600	3200	4800	6400
N_											
NNW											
MM											
WW											
₩											
WSW											
SW											
SSW								3	8	309	29
							19		8		54
SSE								6	21	23	100
SE	·							8	73	148	408
ESE								8	65	159	94
E					-			3	31	96	125
ENE									26	160	145
NE									57	189	189
NNE									205	307	245

Distances 8000 10000 15000 20000 30000 40000 50000 60000 80000 N MNM NW WNW W WSW SW SSW 56 5 107 SSE 109 SE 75 ESE 82 52 E ENE 78 NE 316 NNE 1593

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INSTALLATION: PALISADES

LOCATION: 5 MI S SOUTH HAVEN, MI

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: CONSUMERS PWR

DOCKET NO.: 50-255

THERMAL PWR (MWH): 1.14E+07

COMMERCIAL OPERATION: 12/31/71

LICENSED PWR (MWH): 2.53E+03

NET ELECTRIC PWR (MWH): 3.44E+06

INITIAL CRITICALITY: 05/24/71

COOLING WATER SOURCE: LAKE MICHIGAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	9.10E-01
CR-51	1.74E-05
MN-54	5.71E-06
CO-58	6.88E-05
CO-60	9.62E-05
SE-75	2.42E-04
KR-85	2.65E+00
KR-85M	8.06E-01
KR-87	1.79E+00
KR-88	2.07E+00
SR-89	2.47E-06
SR-90	2.30E-06
NB-95	4.84E-06
I-131	2.02E-02
XE-131M	6.95E-01
I-132	7.1 <b>0</b> E-04
I-133	4.64E-03
XE-133	2.40E+03
XE-133M	7.14E-01
CS-134	1.05E-07
I-135	5.04E-04
XE-135	4.84E+00
XE-135M	6.55E+00
CS-137	<b>9.90E</b> -06
XE-138	4.44E+00
UNIDENTIFIED	2.67E-05

TOTAL AIRBORNE TRITIUM RELEASE 4.17E+00

CAP-88 INFORMATION SHEET Source Category: Reactors Date: April 1991 Palo Verde Units 1 , 2 & 3\_\_\_\_ Facility: Wintersburg, AZ Address: Population Assessment JCL File Name: PALO Array attached Latitude: 33 • 25 • 11• Longitude: 112 52 5 Pop. File Name: PALO Individual Assessment JCL File Name: Distances (meters) <u>200 <u>300</u> <u>400</u> <u>500</u> <u>600</u> <u>800</u> <u>1200</u> <u>1600</u> <u>3200</u> <u>4800</u> <u>6400</u> <u>8000</u> <u>10000</u> <u>15000</u> <u>20000</u> <u>30000</u> <u>40000</u> <u>50000</u> <u>60000</u> <u>80000</u></u> Food Fracil is: F1 F2 F3 Meat \_.oo8 \_.992 .000 Urban Milk .000 1.00 .000 Rural Veg \_076 \_924 \_000 WEATHER DATA: STAR File Name. PNX0309 \_ Array attached <u>231</u>83 Temperature: 22 ℃ WBAN: 309 Header: Lid Height: \_\_\_1800 meters Code: Rainfall: 15 cm/yr PNX Set No: \_\_ STACK Source: 1 45 5 Height (m): Diameter (m): \_ AREA Source Diameter (m<sup>3</sup>): BUOYANT cal/sec \_\_\_\_\_ \_\_\_\_ MOMENTUM m/s \_\_\_\_\_ ENTERED Rise 0 0 0 0 0 0 Pasquill Category: A B C D E F <u>x</u> ENTERED Rise Nuclide Class Diam Ci/vr

C-90

COMMENTE: Phoenix met wats 10-yr 40 miles E of plant

File Name: PALO

					Dist	ances					
	200	300	400	500	600	800	1200	1600	3200	4800	6400
N						<del></del>		7		65	32
NNW		•								24	7
NW										7	
WNW											
W											
WSW											
SW			_								
SSW			_								7
S											
SSE										•	32
SE											
ESE											16
Ε										7	
ENE										7	
NE						•		7		7	7
NNE								7		24	56

	Distances									
	8000	10000	15000	20000	30000	40000	50000	60000	80000	
N	24									
NNW						<b>√</b>				
NW	7									
WNW										
W										
WSW										
SW										
SSW										
S	16						,			
SSE	24									
SE										
ESE										
E										
ENE	24									
NE	24									
NNE	56									

INSTALLATION: PALO VERDE 1" -

LOCATION: 36 MI W OF PHOENIX, ARIZONA

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-528

THERMAL PWR (MWH): 2.03E+07

COMMERCIAL OPERATION: 01/28/86

LICENSEE: ARIZONA NUC. PWR PROJ.

LICENSEE: ARIZONA NUC. PWR PROJ.

LICENSEE: ARIZONA NUC. PWR PROJ.

NET ELECTRIC PWR (MWH): 3.80E+03

NET ELECTRIC PWR (MWH): 6.67E+06

INITIAL CRITICALITY: 05/25/85

COOLING WATER SOURCE: SEWAGE TREATMENT

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CÍ)
AR-41	5.63E-01
MN-54	7.64E-06
CO-58	4.63E-05
FE-59	2.04E-06
CO-60	8.75E-05
BR-82	5.16E-05
KR-85	2.58E+01
KR-85M	4.26E-01
KR-87	7.21E-04
KR-88	3.49E-02
RB-88	4.21E-03
ทั <b>B-9</b> 5	7.33E-06
ZR-95	1.82E-06
RU-103	1.15E-06
SN-117M	1.23E-06
SB-124	2.41E-05
I-131	1.56E-03
XE-131M	6.99E+00
I-133	5.07E-04
XE-133	1.57E+03
XE-133M	6.22E+00
CS-134	2.31E-05
I-135	6.97E-04
XE-135	6.87E+01
CS-137	4.23E-05
CS-138	2.33E-03
XE-138	1.59E+02
BA-140	5.54E-06
CE-144	8.50E-06

TOTAL AIRBORNE TRITIUM RELEASE 4.23E+02

INSTALLATION: PALO VERDE 2

LOCATION: 36 MI W OF PHOENIX, ARIZONA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR

DOCKET NO.: 50-529

THERMAL PWR (MWH): 2.07E+07

COMMERCIAL OPERATION: 09/19/86

LICENSEE: ARIZONA NUC. PWR PROJ.

LICENSED PWR (MWT): 3.80E+03

NET ELECTRIC PWR (MWH): 6.75E+06

INITIAL CRITICALITY: 04/18/86

COOLING WATER SOURCE: SEWAGE TREATMENT

AIRBORNE	<b>EFFLUENTS</b>	
NUCLIDES	RELEASED	ACTIVITY (CI)
AR-41		1.73E+00
CR-51		4.02E-05
MN-54		9.01E-06
CO-58		3.54E-04
CO-60		1.16E-04
BR-82		1.10E-04
KR-85		4.39E+01
KR-85M		1.00E+00
KR-87		1.80E-02
KR-88		2.50E+00
RB-88		4.89E-02
SR-89		5.19E-08
NB-95		5.23E-05
ZR-95		1.93E-05
MO-99		4.26E-08
RU-103		1.13E-05
SB-122		3.97E-07
SB-124		5.56E-04
I-131		4.54E-02
XE-131	M	5.50E+01
I-132		2.76E-06
I-133		6.67E-04
XE-133		2.81E+03
XE-133		. 1.11E+01
CS-134		3.98E-05
I-135		1.14E-05
XE-135		4.16E+01
CS-137		4.93E-05
CS-138		4.36E-04
XE-138		4.04E-02
BA-139		1.95E-06
CE-141		7.19E-05
CE-144		2.52E-08

TOTAL AIRBORNE TRITIUM RELEASE 2.81E+02

-INSTALLATION: PALO VERDE 3

LOCATION: 36 MI W OF PHOENIX, ARIZONA

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 3 TYPE: PWR LICENSEE: ARIZONA NUC. PWR PROJ. DOCKET NO.: 50-530 LICENSED PWR (MWT): 3.80E+03 THERMAL PWR (MWH): 3.03E+07 NET ELECTRIC PWR (MWH): 3.55E+04 COMMERCIAL OPERATION: 01/08/88 INITIAL CRITICALITY: 10/25/87

COOLING WATER SOURCE: SEWAGE TREATMENT

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	7.86E+00
CR-51	9.58E-07
BR-82	8 <b>`.</b> 59E-05
KR-85	6.60E-02
KR-85M	1.09E-01
KR-88	3.08E-02
RB-88	2.20E-02
SB-122	7.59E-07
I-131	1.23E-04
XE-131M	2.43E-01
I-132	6.81E-06
I-133	1.28E-04
XE-133	1.23E+02
XE-133M	1.11E-01
I-135	1.11E-05
XE-135	4.15E+00
CS-138	2.55E-03
XE-138	1.82E-04

TOTAL AIRBORNE TRITIUM RELEASE 3.80E+02

CAP-88 INFORMATION SHEET  Source Category: Reactors	•
Facility: Point Beach Units 1 & 2  Address: Two Rivers, WI	
Population Assessment JCL File Name: POINT  Array attached	
Latitude: 44 16 50 Pop. File Name: POINT	
Individual Assessment	
Distances (meters) 200 300 400 500 600 800 1200 1600 3200 480 6400 8000 10000 15000 20000 30000 40000 50000 60000 800	
Food Fractions: F1 F2 F3  Meat .008 .992 _000 Urban  Milk .000 1.000 _000 Rural	
Veg _076 _924000  WEATHER DATA:  Array attached STAR File Name: ERB0776	
WBAN: 14898 Header: 776 Lid Height: 850 Code: ERB Rainfall: 79 cm/yr Set No:	,
STACK Source: 1 2 3 4 5 6  Height (m):	•
AREA Source . Diameter (m):	
BUOYANT cal/sec	
X ENTERED Rise 0 0 0 0 0 0 0 maters Pasquill Category: A B C D E F G	
Nuclide Class Diam Ci/yr:	

COMMENTS: Green Bay 10-yr met data 35 miles NWof plant

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File Name: POINT

Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											23
NNW								11	22	53	53
MM									11	84	21
WNW								11	32	32	32
W								6	27	37	68
WSW									-32	37	27
SW								11	27	21	32
SSW								6	32	42	53
S								6	11	42	32
SSE								6	58	27	84
SE											
ESE											
E											
ENE											
NE											
NNE											

Distances

	8000	10000	15000	20000	30000	40000	50000	60000	80000
	32								
NNW	46								
NW	79								
WNW	58								
W	58								
WSW	68								
SW	53								
SSW	68								
S	37								
SSE	25	`							
SE	ı								
ESE									
E									
ENE									
NE									
NNE									

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INSTALLATION: POINT BEACH LOCATION: 15 MI N MANITOWOC, WI

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-266

THERMAL PWR (MWH): 1.17E+07

COMMERCIAL OPERATION: 12/21/70

LICENSEE: WISCONSIN ELECTRIC PWR
LICENSED PWR (MWT): 1.52E+03

NET ELECTRIC PWR (MWH): 3.83E+06

INITIAL CRITICALITY: 11/02/70

COOLING WATER SOURCE: LAKE MICHIGAN

UNIT NUMBER: 2 TYPE: P'R

DOCKET NO.: 50-301

THERMAL PWR (MWH): 1.14E+07

COMMERCIAL OPERATION: 10/01/72

LICENSEE: WISCONSIN ELECTRIC PWR
LICENSED PWR (MWT): 1.52E+03

NET ELECTRIC PWR (MWH): 3.72E+06

INITIAL CRITICALITY: 05/30/72

COOLING WATER SOURCE: LAKE MICHIGAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
ALPHA	5.11E-06
F-18	3.87E-05
NA-24	4.29E-04
AR-41	1.96E+00
MN-54	4.86E-05
CO-57	1.13E-08
CO-58	3.59E-05
CO-60	3.64E-04
ZN-65	9.27E-06
KR-85	9.84E-01
KR-85M	7.31E-01
KR-87	1.48E+00
KR-88	1.69E+00
RB-88	4.00E-06
SR-89	3.70E-06
SR-90	4.30E-06
NB-95	6.25E-08
ZR-95	3.56E-09
NB-97	1.60E-08
MO-99	2.71E-08
TC-99M	2.20E-07
AG-110M	2.31E-07
SN-113	4.80E-10
SB-125	9.39E-08
I-131	5.43E-04
I-132	4.78E-04
TE-132	3.07E-06
I-133	1.53E-03
XE-133	6.04E+01
XE-133M	3.35E-01
CS-134	7.27E-05
I-135	9.18E-05
XE-135	3.53E+00
XE-135M	2.54E+00

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### INSTALLATION: POINT BEACH

CS-137	6.74E-04
CS-138	1.26E-07
XE-138	7.19E+00

### TOTAL AIRBORNE TRITIUM RELEASE 1.26E+02

CAP-88 INFORMATION SHEET

Date: <u>*pril 1991</u>	Source Categ	ory: Re-	ctors
Facility: Prairie I land 1 & 2  Address: Red Wing, MN			
Population Assessment JCL File Array attached	Name: _	PRAIRIE	
Latitude: 44 * 37 · 18 *	ile Name: _		
Individual Assessment JCL File	Name: _	<del></del>	
Distances (meters) 200 300 400 500 600 6400 -8000 10000 15000 20000	800 120 30000 4000		
Food Fractions: F1 F2 F3  Meat .008 .992 .000  Milk .000 1.00 .000	;	Irban Rural	
WEATHER DATA:  Array attached STAF	R File Name:	MSPO26	7
WBAN: 14822 Temperature: Header: 267 Lid Height: Rainfall: Set No:	7 °C 		
STACK Source: 1 2 Height (m): 60 Diameter (m):	3 =====================================	<u> </u>	5 6
AREA Source Diameter (m ):			
BUOYANT cal/sec MOMENTUM m/s			
ENTERED Rise Pasquill Category: A B C	D E	F G	meters
Nuclide Class Diam Ci/vr:			
COMMENTS: Minneapolis/St. Prul 5-	yr met data	40 mıle	s NW of pl.nt

File Name: PRAIRIE

Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N									133	18	. 32
NNW										5	14
NW								34			4
WNW							0	9	25	25	18
W		_						25	25	13	22
WSW								17		30	43
SW								4		43	43
SSW						•		4		25	<u>56`</u>
s										18	34
SSE							13		4	9_	69
SE									13		
ESE									5	5	102
E									11	42	37_
ENE								7		23	23
NE									23	18	32
NNE									91	18	42

Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	32								
NNW	14								
NW	9								
WNW	22								
W	18								
WSW	25								
SW	25								
SSW	22								
S	81								
SSE	510								
SE	51								
ESE	120								
E	37								
ENE	_60								
NE	55								
NNE	42								

INSTALLATION: PRAIRIE ISLAND

LOCATION: 26 MI SE MINNEAPOLIS, MN

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-282

THERMAL PWR (MWH): 1.23E+07

COMMERCIAL OPERATION: 12/16/73

LICENSEE: NORTHERN STATES PWR

LICENSED PWR (MWT): 1.65E+03

NET ELECTRIC PWR (MWH): 3.82E+06

INITIAL CRITICALITY: 12/01/73

COOLING WATER SOURCE: MISSISSIPPI RIVER

UNIT NUMBER: 2 TYPE: PWR

DOCKET NO.: 50-306

THERMAL PWR (MWH): 1.26E+07

COMMERCIAL OPERATION: 12/21/74

LICENSEE: NORTHERN STATES PWR
LICENSED PWR (MWT): 1.65E+03

NET ELECTRIC PWR (MWH): 3.89E+06

INITIAL CRITICALITY: 12/17/74

COOLING WATER SOURCE: MISSISSIPPI RIVER

AIRBORNE	EFFLUENTS	
NUCLIDES	RELEASED	ACTIVITY (CI)
K-40		1.88E-05
CO-5.8		3.79E-08
CO-60		2.22E-06
KR-85		1.23E-01
SR-89		6.60E-05
SR-90		2.40E-06
CD-109		4.81E-06
I-131		1.20E-06
I-133		2.00E-05
XE-133		1.89E-02
XE-1331	<b>M</b> -	6.36E-06
XE-135		5.14E-04
CS-137		6.97E-07

TOTAL AIRBORNE TRITIUM RELEASE 1.51E+02

N/A=NOT APPLICABLE N/D=NOT DETECTED

N/R=NOT REPORTED

Date: APR.	IL 1991	P-88 INFOR	MATION	SHEET Source (	Category:	Reactors	3
Facility:	R.E.					-	
Address:	Ontar	lo, NY					
	lation Assessm Array attache	d	JCL File	Name:	_GIN	<u> </u>	<del></del>
Latitu Long	itude: $\frac{43}{77}$	18. 32	Pop. F	ile Name:	GIN	NA	
Indivi	dual Assessme	int	JCL File	Name:			
Distances (1 200 6400 8	300 400	500 15000	600 20000	800 30000	1200		3200 4800 0000 80000
Food Fracti	Meat Milk	F1 F2 .008 .99 .000 1.0 .076 .92	000.0	=	_ Urban _ Rural		
WEATHER	_				ne: ROCO	598	
WBAN: Header: Code: Set No:	14768 598		perature: leight: iall:	950	C meters cm/yr		
Heigh	CK Source: nt (m): eter (m):	42	2	3	4	5	6
	A Source eter (m²):	<del></del> .				•	
BUO	YANT cal/sec IENTUM m/s						
X ENTI	ERED Rise uill Category:	<u>A</u> B	<u>C</u>	<b>-</b> P	0 F	0 met	ers
Nuclide	Class Diam	Ci/vr:					

COMMENTS: Rochester 10-yr met asta 15 miles W of plant

File Name: GINNA Distances

	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											
NNW											
NW											
WNW											
W								168	285	560	262
WSW							39		145	85	312
SW						55			55	200	220
SSW					19				55	295	860
s				20					125	135	905
SSE					7				40	350	1686
SE					17				110	670	203
ESE							28 -		40	153	156
E							18		250	291	263
ENE	,										
NE											
NNE											

#### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW									
NW									
WNW									
W	345								
WSW	842								
SW	1004								
SSW	500								
S	428								
SSE	1246								
SE	359								
ESE	275								
E	326								
ENE									
NE									
NNE									

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INSTALLATION: R.E.GINNA

LOCATION: 16 MI NE ROCHESTER, NY

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: ROCHESTER G&E

DOCKET NO.: 50-244 LICENSED PWR (MWT): 1.52E+03
THERMAL PWR (MWH): 1.10E+07 NET ELECTRIC PWR (MWH): 3.53E+06
COMMERCIAL OPERATION: 07/01/70 INITIAL CRITICALITY: 11/08/69

COOLING WATER SOURCE: LAKE ONTARIO

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
C-14	1.77E+00 ~
AR-41	1.19E+00 ~
KR-85M	9.83E-02
KR-87	1.50E-01
KR-88	2.15E-01
I-131	4.33E-05
XE-131M	1.97E-01
·I-133	2.80E-03
XE-133	3.46E+01
XE-133M	2.82E-02
XE-135	1.26E+01
XE-135M	1.77E+00
XE-138	6.48E-01
UNIDENTIFIED	1.36E-05

#### TOTAL AIRBORNE TRITIUM RELEASE 1.68E+02

**CAP-88 INFORMATION SHEET** Source Category: Reactors --Date: April 1991 Rancho Seco Facility: Clay Station, CA Address: RANCHO Population Assessment JCL File Name: Array attached 38 • 20 · 39 • Latitude: 121° 7' 12° Pop. File Name: RANCHO Longitude: Individual Assessment JCL File Name: Distances (meters) 600 400 500 800 300 1200 1600 4800 200 10000 15000 20000 30000 40000 50000 6400 8000 60000 80000 Food Fractions: F1 F2 F3 Meat .008 .992 .000 Urban 000 1.00 .000 Milk Rural Veg WEATHER DATA: STAR File Name: \_\_\_SAC0320 Array attached 23232 14 WBAN: Temperature: \_\_\_ C \_600 meters Header: Lid Height: 320 Code: Rainfall: 43 \_ cm/yr Set No: STACK Source: 46 46 2 3 5 Height (m): Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec \_\_\_ MOMENTUM m/s X ENTERED Rise O O O O O Meters Pasquill Category: Nuclide Class Diam Ci/vr:

COMMENTS: Sacramento 5-yr met data

. File Name: RANCHO

### Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
								4			
NNW									<u>.</u>		
NW											4
WNW											10
W								5	5	30	10
WSW									80	32	10
SW										75	
SSW											
5									0		
SSE											
SE											
ESE								4			
E								4			
ENE								4			
NE								7	``	7	
NNE								4			

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW	8								
NW	10								
WNW	4								
W	45								
WSW	10								
SW	20								
SSW	12								
s	8								
SSE	4								
SE	8								
ESE									
E									
ENE									
NE									
- NNE									

--- INSTALLATION: RANCHO SECO

LOCATION: 25 MI SE SACRAMENTO, CA

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR
DOCKET NO.: 50-312
THERMAL PWR (MWH): 9.47E+06
COMMERCIAL OPERATION: 04/17/75
LICENSEE: SACRAMENTO MUN. UTILITY
LICENSED PWR (MWT): 2.77E+03
NET ELECTRIC PWR (MWH): 2.81E+06
INITIAL CRITICALITY: 09/16/74

COOLING WATER SOURCE: FOLSOM CANAL

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	2.37E-02
MN-54	2.18E-06
CO-60	3.40E-05
KR-85	2.01E+00
KR-85M	2.95E+00
SR-85	1.58E-06
KR-87	4.84E-03
KR-88	2.38E+00
RB-88	1.88E-04
I-131	1.98E-04
XE-131M	1.53E+01
I-133	4.19E-05
XE-133	1.46E+03
XE-133M	9.66E+00
XE-135	2.53E+01
XE-135M	1.99E-04
CS-137	7.83E-06

TOTAL AIRBORNE TRITIUM RELEASE 1.75E+01

N/A=NOT APPLICABLE N/D=NOT DETECTED

N/R=NOT REPORTED

## **CAP-88 INFORMATION SHEET**

Date: _April 1991	Source Category: <u>Reactors</u>	
Facility: Srlem 1 & 2 Address: Salem, NJ		
Population Assessment	Name: SALEM	
Latitude: 39 27 36 Longitude: 75 72 8 Pop. F	ile Name: SALEM	
Individual Assessment	Name:	
Distances (meters)         400         500         600           6400, 8000         10000         15000         20000	800 1200 1600 3200 4806 30000 40000 50000 60000.	
Food Fractions: F1 F2 F3  Meat .008 .992 .000  Milk .000 1.00 .000  Veg .076 .924 .000	Urban Rural	
WEATHER DATA:	R File Name:ILC1058	
WBAN: 13791 Temperature: Header: 1058 Lid Height: Code: ILC Rainfall: Set No:	12 °C 600 meters 107 cm/yr	
STACK Source: 1 2 Height (m): 59 59 Diameter (m):	3 4 5 6	
AREA Source Diameter (m):		
BUOYANT cal/sec MOMENTUM m/s		
X ENTERED Rise 0 0 0 Pasquill Category: A B C	0 0 0 0 meters	
Nuclide Class Diam Ci/yr:		
COMMENTS: "llmington, DL 5-yr me	t data 15 miles N of pl nt	

C-108

File Name: SALEY

	Distances										
	200	300	400	500	600	800	1200	1600	3200	4800	640C
N					•						
NNW											10
NW											108
WWW											113
W											22
WSW											
SW											
SSW											16
S											
SSE										_	
SE											
ESE											
E											3
ENE						•				Õ	35
NE											9
NNE											

					Dis	tances			
	8000	10000	15000	20000			50000	60000	80000
N									
MNW	262								
NW	104								
MNW	58								
₩	54								
WSW	31								
SW	15								
SSW	11								
S	16								
SSE									
SE								•	
ESE									
E									
ENE	96								
NE	295								
NNE	33								

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INSTALLATION: SALEM 1

LOCATION: 20 MI S WILMINGTON, DE

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: PUB.SER. E&G OF NJ DOCKET NO.: 50-272 LICENSED PWR (MWT): 3.34E+03 NET ELECTRIC PWR (MWH): 7.41E+06 COMMERCIAL OPERATION: 06/30/77 INITIAL CRITICALITY: 12/11/76

COOLING WATER SOURCE: DELAWARE RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	7.51E-03
CO-58	3.66E-05
CO-60	1.42E-03
KR-85	1.10E+00
KR-85M	3.58E-01
I-131	6.73E-04
XE-131M	7.85E+00
XE-133	5.00E+02
XE-133M	9.90E-01
XE-135	1.90E+01
XE-135M	2.37E-04
CS-137	5.69E-06

TOTAL AIRBORNE TRITIUM RELEASE 4.00E+02

INSTALLATION: SALEM 2

LOCATION: 20 MI S WILMINGTON, DE

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR LICENSEE: PUB. SER. E&G OF NJ DOCKET NO.: 50-311 LICENSED PWR (MWT): 3.34E+03 NET ELECTRIC PWR (MWH): 5.97E+06 COMMERCIAL OPERATION: 10/13/81 INITIAL CRITICALITY: 08/08/80

COOLING WATER SOURCE: DELAWARE RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	1.55E-02
CO-57	1.99E-07
CO-58	7.05E-05
CO-60	2.72E-05
KR-85	1.82E+00
KR-85M	3.08E+00
KR-87	5.66E-01
KR-88	4.15E+00
TE-125M	6.82E-06
I-131	8.80E-04
XE-131M	2.65E+00
XE-133	1.12E+03
XE-133M	6.08E+00
XE-135	3.43E+01
XE-135M	4.99E+00
CS-137	6.66E-06
CE-141	1.25E-07

TOTAL AIRBORNE TRITIUM RELEASE 3.70E+02

## CAP-88 INFORMATION SHEET

Date: April	1991			Source Ca	ategory:	Reacto	rs	<del>-</del> -
Facility:	S n Onofre San Clemen		1, 2 &	3				· .
/	ion Assessm Arrav attached	1			SAN	ONOFER	-	•
Latitude Longitu	de: $\frac{11}{117}$	23' 13° 33' 24°	Pop. F	le Name:	SAN	ONOFER		•
Individu	al Assessme	nt	JCL File	Name:			<del></del>	•
Distances (me 200 300 6400 8000	_400_	500 15000	600 20000	800 30000	1200 40000		3200 60000	4800 80000
Food Fraction	Meat .	1 F2 008 _99 000 1.0	2_000	<del></del>	_ Urban _ Rural			
WEATHER DA	Veg .	076 <u>.924</u>	-000	R File Nam	e: <u>LGB1</u>	052		•
	1052	Tem Lid I Rain	leight:	19 0 600 m	eters			
STACK Height Diamete	• •	63	<b>2</b> 63	3	4	5	6	<del></del>
AREA Diamete		<del></del>						<del></del>
MOME	NT cal/sec NTUM m/s					<del></del>		
ENTER Pasquil	ED Rise I Category:	0 0 A B	<u>C</u>	0 0 E	- <del>  0</del>	G me	eters	
Nuclide	Class Diam	<u>Ci/yr:</u>						<u>-</u>

COMMENTS: Long Beach met data 30 miles NNW of plant

					D1:	stance	5				
	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											2564
NNW									945	810	
, NW									865	81	3157
MNW										3	
W											
WSW											
SW											
SSW											
S											
SSE											
SE											
ESE											
E											
ENE										3	
NE										480	4
NNE											

### Distances

			<b></b>						
_	8000	10000	15000	20000	30000	40000	50000	50000	80000
N	730								
NNW	4612								
NW									
WNW									
₩									
WSW									
SW									
SSW									
S									<del>-</del>
SS€									
SE									
ESE									
Ε	2426								
ENE									
NE									
NNE									

INSTALLATION: SAN ONOFRE 1 LOCATION: 2.5 MI S SAN CLEMENTE, CA

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: SOUTHERN CAL EDISON &

SAN DIEGO G&E

LICENSED PWR (MWT): 1.35E+03 **DOCKET NO.: 50-206** THERMAL PWR (MWH): 4.52E+06 THERMAL PWR (MWH): 4.52E+06 NET ELECTRIC PWR (MWH): 1.37E+06 COMMERCIAL OPERATION: 01/01/68 INITIAL CRITICALITY: 06/14/67

COOLING WATER SOURCE: PACIFIC OCEAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	5.79E-03
CR-51	1.05E-06
MN-54	2.21E-06
CO-58	6.46E-05
.CO-60	5.48E-06
BR-82	1.01E-04
KR-85	1.15E+01
KR-85M	3.70E+00
KR-87	2.54E-01
KR-88	1.15E+00
RB-88	2.48E-03
SR-89	3.41E-07
SR-90	1.26E-08
I-131	1.03E-02
XE-131M	3.92E+00
I-132	3.88E-03
I-133	1.07E-03
XE-133	2.85E+03
XE-133M	2.67E+01
CS-134	1.91E-04
I-135	5.34E-04
XE-135	8.83E+01
XE-135M	1.12E-03
CS-137	2.41E-04
CS-138	5.87E-03
BA-140	4.52E-07

TOTAL AIRBORNE TRITIUM RELEASE 2.05E+01

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR LICENSEE; SOUTHERN CAL EDISON &

SAN DIEGO G&E

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DOCKET NO.: 361 LICENSED PWR (MWT): 3.41E+03

THERMAL PWR (MWH): 2.75E+07 NET ELECTRIC PWR (MWH): 9.00E+06 COMMERCIAL OPERATION: 08/08/83 INITIAL CRITICALITY: 07/26/82

COOLING WATER SOURCE: PACIFIC OCEAN

UNIT NUMBER: 3 TYPE: PWR LICENSEE: SOUTHERN CA EDISON&SAN

DIEGO GAS&ELEC

DOCKET NO.: 362 LICENSED PWR (MWT): 3.39E+03

THERMAL PWR (MWH): 1.89E+07 NET ELECTRIC PWR (MWH): 6.13E+06

COMMERCIAL OPERATION: 04/01/84 INITIAL CRITICALITY: 08/29/83

COOLING WATER SOURCE: PACIFIC OCEAN

AIRBORNE E		
	ELEASED	ACTIVITY (CI
NA-24		4.71E-06
AR-41		1.73E+01
CR-51		1.14E-04
MN-54		1.14E-04
CO-57		8.38E-11
CO-58		1.31E-03
CO-60		6.08E-04
BR-82		1.55E-04
KR-85		1.60E+01
KR-85M		3.26E+01
KR-87		6.04E+00
KR-88		8.55E+00
RB-88		2.51E-02
SR-89		7.22E-08
SR-90		2.99E-09
SR-92		1.01E-06
Y-92		1.77E-06
NB-95		1.69E-04
NB-95M		3.16E-06
ZR-95		3.28E-05
NB-97		7.24E-04
MO-99		1.12E-05
`TC-99M		1.14E-05
RU-103		1.22E-06
SN-113		3.51E-06
I-131		7.49E-02
XE-131M		1.54E+01
I-132		4.50E-04
TE-132		2.49E-07
I-133		1.62E-02
XE-133		4.69E+03
XE-133M		6.34E+00

## INSTALLATION: SAN ONOFRE 2-3-

6.69E-06
1.87E-03
3.18E+02
3.23E+00
2.94E-07
2.44E-04
4.83E-03
9.86E-01
1.23E-05

### TOTAL AIRBORNE TRITIUM RELEASE 1.89E+01

## CAP-88 INFORMATION SHEET

Date:May_1991	Source Category: _	Reactors
Facility: Sequovah Units 1 and 2  Address: Daisy, TN	····	<del></del>
Population Assessment		EOUOYAH
Latitude: 35 13 23" Longitude: 85 5 16" Pop. F	ile Name:s	SEQUOYAH
Individual Assessment JCL File	Name:	
Distances     (meters):       200     300     400     500     600       6400     8000     10,000     15,000     20,000		600 3200 4800 2,000 60,000 80,000
Food Fractions: F1 F2 F3 Meat <u>.442</u> <u>.558</u> .000	Urban	
WEATHER DATA: Milk .399 .601 .000 .000	Rural R File Name:CHA	0711
WBAN13882 Temperature: Header:711 Lid Height: Code: Rainfall. Set No:		•
STACK Source: 1 2 Height (m): 40 Diameter (m):	3 4	5 6
AREA Source Diameter (m²):		
BUOYANT cai/sec MOMENTUM m/s		
x ENTERED Rise 0 0 0 Pasquill Category: A B C	0 0 0 D E F	o meters G
Nuclide Class Diam Ci/yr:		
COMMENTS: Chattanoogo/Lovel 5-year Me	t Data approximately	15 miles SSW of plant

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File Name: SEQUOYAH

### Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
X										30	20
NNW							5		210	25	40
NW								35	110	260	125
WNW									50	225	555
W							20		60	105	310
WSW							3		90	570	470
Sk									1	25	265
SSW									165	155	390
S			`	-					150	15	270
SSE									125	165	570
SE								20	190	30	205
ESE								30	70	185	300
E								3	45	5	215
ENE									45	10	140
NE									3	20	30
NNE										45	150

#### Distances

	8000	10000	15000	20000	30000	40000	50000	60000	80000
	20								
NNA	55								
Nk	165								
WNW	440								
W	360								
WSW	1220								
SW	100								
SSW	230								
S	<b>33</b> 5								
SSE	280								
SE	180								
ESE	190								
E	150								
ENE	230								
NE	10								
NNE	90								

- INSTALLATION: SEQUOYAH LOCATION: DAISY, TN

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1983 AIRBORNE EFFLUENTS

UNIT NUMBER 1 TYPE. PWR LICENSEE: TENNESSEE VALLEY AUTHORITY

DOCKET NO.: 50-327 LICENSED POWER (MWT): 2.82E+03

THERMAL POWER (MWH): 5 16E+05 NET ELECTRIC POWER (MWH): 6 71E+04

COMMERCIAL OPERATION. 07/01/81 INITIAL CRITICALITY: 07/05/80

COOLING WATER SOURCE: TENNESSEE RIVER

UNIT NUMBER: 2 TYPE: PWR LICENSEE. TENNESSEE VALLEY AUTHORITY

DOCKET NO.. 50-328 LICENSED POWER (MWT). 2.82E+03

THERMAL POWER (MWH): 1.26E+07 NET ELECTRIC POWER (MWH): 3.88E+06

COMMERCIAL OPERATION: 06/01/82 INITIAL CRITICALITY: 11/05/81

COOLING WATER SOURCE: TENNESSEE RIVER

ATRBORNE	<b>EFFLUENTS</b>
VIIIOOIGIE	THE PUBLICA

NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	3.50E+00
CR-51	4.59E-05
MN - 54	9.15E-06
CO-58	3.35E-04
CO-60	4.10E-06
KR-85	8.44E+00
KR-85M	4.55E+00
KR-87	2.96E+00
KR-88	4.59E+00
SR-89	7.32E-04
SR-90	9.89E-08
Y-91	1.63E-04
NB-95	2 67E-06
RU-106	2.66E-04
I-131	6 64E-04
XE-131M	6 53E+00
I-133	2.54E-04
XE-133	3.63E+03
XE-133m	3.63E+01
1-135	2.22E-04
XE-135	2.18E+02
XE-135M	3.70E+00
CS-137	1.43E-09
XE-138	1.68£+00

TOTAL AIRBORNE TRITIUM RELEASE

8 46E+02

N/A=NOT APPLICABLE N/D=NOT DETECTED

N/R-NOT REPORTED

Date: Arr	11 1991 CA	VP-88 INFO	RMATION	Source Ca	ategory:	Reactor	: ° ?	
Facility:	Shearon H	arris Un	t 1			<del></del>		
Address:	Bonsal. N	C					<del></del>	
	ation Assessr Array attach		JCL File	Name:	HARR	rs		
	de: _35° tude: _79°		Pop. F	ile Name:	HARI	ris		
Individ	dual Assessm	ent	JCL File	Name:				
Distances (n 200 3 6400 8	neters) 00 400 000 10000	<u>500</u> 15000	<u>600</u> 20000	800 30000	1200 40000	1600 50000		800 80000
Food Fraction	ons:		. –		Urban			
WFATHER I	Milk Veg	.399 .600 .700 .300	.000		Rural			
	Array attach	ed	STA	R File Name	e:1	RDU0083		
WBAN: Header: Code: Set No:	13722 83 RDU	Lid 1	perature: Height: fall:	15 0 900 m 106 cr	eters			
Heigh	K Source: at (m): eter (m):	40	2	3	4	5	6	- · -
	Source (m*):							-
мом	(ANT cal/sec ENTUM m/s							<u>-</u>
X ENTE	RED Rise uill Category:	$\frac{0}{A}$ $\frac{0}{B}$	- <u>°</u>	0 D E	<u>0</u>	G met	ters	
Nuclide ———	Class Diam	Ci/yr:						<u>-</u> -
								_
COMMENTS	S: Raleigh	5-yr met	aata	20 miles	S E OÎ	prent		

C-120

File Name: HARRIS

N NNW	300	400	500	600	800	1200	1600	3200 32	4800 38 114	- 47 - 20
NNW								32		
								32	11/	20
A.H.J									1 1-4	29
NW								1047	22	29
WNW									19	29
₩									17	29
wsw										29
SW									5	29
SSW										29
S										32
SSE										68
SE										71
ESE									22	89
Ε							•	3	10	. 80
ENE									4	92
NE									60	55
NNE								3	50	<b>5</b> 5

		Distances							
_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N_	55							*	
NNW	38								
NW	38								
WNW	38								
₩	38								
WSW	38								
SW	38								
SSW	38								
S	38								
SSE	56								
SE	115								
ESE	124								
E	126								
ENE	96								
NE	71								
NNE	71								

INSTALLATION: HARRIS

LOCATION: 20 MI SW RALEIGH, NC

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: CAROLINA P&L

DOCKET NO.: 50-400 LICENSED PWR (MWT): 2.77E+03
THERMAL PWR (MWH): 1.71E+07 NET ELECTRIC PWR (MWH): 5.33E+06
COMMERCIAL OPERATION: 05/02/87 INITIAL CRITICALITY: 01/03/87

COOLING WATER SOURCE: MAKEUP RESERVOIR

AIRBORNE	<b>EFFLUENTS</b>	
NUCLIDES	RELEASED	ACTIVITY (CI)
<b>ALPHA</b>		1.03E-07
AR-41		3.55E-01
CR-51		6.04E-06
CO-58		3.45E-05
CO-60		3.96E-06
KR-85M		4.86E+01
KR-87		1.62E+01
KR-88		' 8.10E+01
NB-95		1.48E-06
<b>I-132</b>		3.40E-06
XE-133		1.94E+03
XE-1331	1	3.24E+01
XE-135		1.13E+02
XE-138		1.62E+01

#### TOTAL AIRBORNE TRITIUM RELEASE N/D

N/A=NOT APPLICABLE N/D=NOT DETECTED

N/R=NOT REPORTED .

CAP-88 INFORMATION SHEET Date: May 1991 Source Category: Reactors South Texas Project Units 1 and 2 Facility: Address: Palacious, TX Population Assessment JCL File Name: SOUTH Array attached Latitude: 28 \* Pop. File Name: Longitude: 47 " 96 • 12 • SOUTH Individual Assessment JCL File Name: Distances (meters): 200 400 500\_ 600\_ 800 1200 1600 3200 4800 300 6400 8000 15,000 20,000 30,000 40,000 50,000 60,000 80,000 10,000 Food Fractions: F1 F2 F3 .992 .000 Meat .008 Urban .000 .000 1.000 Milk Rural Veg .076 .924 .000 WEATHER DATA: \_\_\_ Array attached STAR File Name: \_\_\_\_\_CRP1459 WBAN: 12924 Temperature: \_\_\_\_20 Heacler: 900 Lid Height: meters Code: Rainfall: 114 \_cm/yr Set No: STACK Source: 1 2 3 4. 5 6 Height (m): 31 Diameter (m): AREA Source Diameter (m<sup>2</sup>): MOMENTUM m/s 0 0 0 0 0 0 0 meters

A B C D E F G X ENTERED Rise Pasquili Category: Class Diam Ci/vr. Nuclide

COMMENTS: Corpus Christi 5-year Met Data approximately 90 miles SW of plant

File Name: SOUTH

Distances 1

							_				
_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N									0	46	
NNW									_		
NW											
WNW						-	0			10	
W										15	10
WSW											
SW											
SSW	0										10-
S											
SSE		-									
SE					_					4	230
ESE									4		230
Е		-									
ENE			<u>-</u>	<u> </u>							
NE ,											
NNE . /	•							·			

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N			-						
NNW									
NW	87								
WNW	46								
W	37								
WSW	4								
SW	29							· .	
SSW									
S									
SSE									
SE	230								
ESE	228					_			
E	4								
ENE									
NE									
NNE									

INSTALLATION: SOUTH TEXAS 1 & 2

LOCATION: 12 MI-SSW OF BAY CITY, TX .

#### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1990 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: HOUSTON LIGHTING & POWER THERMAL POWER (MWH): 8.81E+06

COMMERCIAL OPERATION: 08/25/88

LICENSED POWER (MWT): 3.82E+03

NET ELECTRIC POWER (MWH): 2.79E+06

INITIAL CRITICALITY: 03/08/88 LICENSED POWER (MWT): 3.82E+03 DOCKET NO.: 50-498

COOLING WATER SOURCE: MAIN COOLING RESERVOIR

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
CR-51	1.20E-04
MN - 54	4 41E-06
GO-58	3.05E-04
FE-59	3.42E-06
CO-60	3 43E-05
KR-85M	5 53E+02
ZR-95	3.98E-06
SB-124	1.16E-07
KR-87	3.65E+02
KR-88	5.21E+02
RB-88	4.02E-03
RB-89	3.87E-04
SR-89	1.90E-03
SR-90	7.19E-06
SR-91	2 05E-03
Y-91M	1.35E-02
TC-99M	2.46E-06
I-131	6.89E-03
XE-131M	8.22E+01
I-132	3.47E-05
TE-132	2.34E-06
I-133	2.54E-02
XE-133	6 96E+03
XE-133M	7 02E+01
CS-134	1 24E-06
1-135	1.66E-02
XE-135	1.30E+03
XE-135M	4.21E+02
CS-137	4.03E-04
CS-138	9.76E-02
XE-138	5 75E+02
BA-139	1.80E-02
BA-140	2.55E-03
LA-140	1 20E-03

TOTAL AIRBORNE TRITIUM RELEASE 4 59E+00

CAP-88 INFORMATION SHEET

Date: _ <u>April 1991</u>	Source Category: Reactors
Facility: St Lucie 1 & 2  Address: Ft. Pierce, FL	
Population Assessment JCL File	Name: SAINT
Latitude: 27 20 54 Longitude: 80 14 47 Pop. F	ile Name: SAINT
Individual Assessment JCL File	Name:
Distances (meters)       200     300     400     500     600       6400     8000     10000     15000     20000	800 1200 1600 3200 4800 30000 40000 50000 60000 80000
Food Fractions: F1 F2 F3 .000 Milk .000 1.00 .000 Veg .076 .324 .000	Urban Rurai
WEATHER DATA:	R File Name:PBI0054
WBAN: Temperature: Header: Lid Height: Code: PBI Rainfall: Set No:	24 C 1200 meters 146 cm/yr
STACK Source: 1 2 Height (m): 62 62 Diameter (m):	3. 4 5 6
AREA Source Diameter (m²):	
BUOYANT cal/sec MOMENTUM m/s	
X ENTERED Rise 0 0 0 Pasquill Category: A B C	$\frac{0}{D}$ $\frac{0}{E}$ $\frac{0}{F}$ $\frac{0}{G}$ meters
Nuclide Class Diam Ci/yr:	

COMMENTS: west Palm met data 1-yr 30 miles S of pl nt

File Name: SAINT

Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											
NNW			,						•		<u>432</u>
NM			•								82
MMM										100	864
W									35_	13	857
WSW									111		538
SW									_30_	59	5175
SSW										93	343
s											156
SSE								·	1381	882	1321
SE									762		
ESE											
E											
ENE			-								
NE											
NNE								_			

Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW									
NW	109								
WNW	3029								
W	2308								
WSW	3884								
SW	4773								
SSW	2567								
S	370								
SSE	1973								
SE									
ESE									
E									
ENE									
NE									
NNE									

INSTALLATION: ST. LUCIE 1

LOCATION: 8 MI S FT. PIERCE, FL

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: FLORIDA P&L DOCKET NO.: 50-335 LICENSED PWR (MWT): 2.56E+03 THERMAL PWR (MWH): 1.97E+07 NET ELECTRIC PWR (MWH): 6.25E+06 COMMERCIAL OPERATION: 12/21/76 INITIAL CRITICALITY: 04/22/76

COOLING WATER SOURCE: ATLANTIC OCEAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	3.04E+00
KR-85M	5.27E+00
KR-87	1.67E-01
KR-88	6 71E+00
I-131	6.40E-03
XE-131M	1.17E-02
I-132	4.28E-02
I-133	4.28E-02
XE-133	1.20E+03
XE-133M	5.47E+00
XE-135	2.03E+02

TOTAL AIRBORNE TRITIUM RELEASE 1.46E+02

INSTALLATION: ST. LUCIE 2 LOCATION: 8 MI S FT. PIERCE, FL

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNL EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR LICENSEE: FLORIDA P&L

DOCKET NO.: 50-389

THERMAL PWR (MWH): 2.35E+07

COMMERCIAL OPERATION: 08/08/83

LICENSED PWR (MWT): 2.70E+03

NET ELECTRIC PWR (MWH): 7.41E+06

INITIAL CRITICALITY: 06/02/83

COOLING WATER SOURCE: ATLANTIC OCEAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	1.88E+00
KR-85	9.60E-01
KR-85M	7.45E+01
KR-87	3.71E-01
KR-88	3.19E+01
RB-88	1.27E+00
I-131	2.85E-02
XE-131M	3.47E+00
I-133	3.89E-02
XE-133	8.01E+03
XE-133M	8.70E+01
XE-135	9.50E+02
CS-137	3.04E-05

#### TOTAL AIRBORNE TRITIUM RELEASE 8.55E+01

Date: April 1991 CAP-88 INFORMATION SHEET Reactors									
Facility: Surry Units 1 & Gravel N-ck. VA									
Population Assessment		SURRY							
Latitude: 37 9 56 Longitude: 76 41 53	' Pop. File Name:	SURRY							
Individual Assessment	JCL File Name:								
Distances (meters)  200 300 400 500  6400 8000 10000 15000		900 1600 3200 4800 900 50000 60000 80000							
Food Fractions: F1 F2 Meat .008 .992 Milk .000 1.00	2 .000	Urban Rural							
WEATHER DATA:  Array attached	STAR File Name:	GVE0824							
WBAN:         13732         Term           Header:         824         Lid I           Code:         EVE         Rain           Set No:	perature: 14 C Height: 800 met fall: 112 cm/								
STACK Source: 1 Height (m):43 Diameter (m):	2 3	4 5 6							
AREA Source Diameter (m):									
BUOYANT cal/sec MOMENTUM m/s									
X ENTERED Rise 0 0 B	C D E	C O meters							
Nuclide Class Diam Ci/yr									

COMMENTS: Gordonville 5-yr met data

File Name: SURRY

### Distances

	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											
NNW											28
NW											
WNW											
W											41
WSW											17
SW											
SSW										48	
S										21	48
SSE											•
SE											
ESE											
E											
ENE											
NE											
NNE											

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW	21								
NW									
WNW									
W	116								
WSW	48								
SW	72					•			
SSW	65								
S	85								
SSE	34								
SE									
ESE									
E	14								,
ENE					,				
NE	6								
NNE	6								

INSTALLATION: SURRY

LOCATION: 19 MI NW NEWPORT NEWS, VA

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-280

THERMAL PWR (MWH): 8.45E+06

COMMERCIAL OPERATION: 12/22/72

LICENSEE: VIRGINIA ELECTRIC & PWR
LICENSED PWR (MWT): 2.44E+03

NET ELECTRIC PWR (MWH): 2.69E+06

INITIAL CRITICALITY: 07/01/72

COOLING WATER SOURCE: JAMES RIVER

UNIT NUMBER: 2 TYPE: PWR

DOCKET NO.: 50-281

THERMAL PWR (MWH): 1.16E+07

COMMERCIAL OPERATION: 05/01/73

LICENSEE: VIRGINIA ELECTRIC & PWR
LICENSED PWR (MWT): 2.44E+03

NET ELECTRIC PWR (MWH): 3.57E+06

INITIAL CRITICALITY: 03/07/73

COOLING WATER SOURCE: JAMES RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	1.22E+00
CR-51	1.45E-04
MN-54	1.11E-04
CO-57	8.60E-06
CO-58	5.22E-03
CO-60	1.90E-03
\$E-75	5.65E-08
KR-85	4.13E+00
KR-85M	1.88E-01
KR-87	9.31E-03
KR-88	1.50E-02
RB-88	2.16E-03
SR-89	1.68E-06
SR-90	2.59E-07
NB-95	7.56E-06
AG-110M	2.63E-10
SB-125	2.04E-05
I-131	9.57E-03
XE-131M	.1.97E+00
I-132	1.09E-03
TE-132	2.38E-06
I-133	4.70E-03
XE-133	3.53E+02
XE-133M	6.03E-01
CS-134	8.79E-04
I-134	9.19E-06
I-135	4.17E-04
XE-135	4.56E+00
XE-135M	6.41E-03
CS-137	2.33E-03
CS-138	6.24E-04

TOTAL AIRBORNE TRITIUM RELEASE 2.79E+01

\_ INSTALLATION: SURRY

Date:April 1991 Source Category: Reactors	
Facility: Three Mile Island 1  Address: Middletown PA	
Population Assessment JCL File Name: TXI Array attached	
Latitude: 40 9 11 Pop. File Name: TMI	
Individual Assessment	
Distances (meters)         600         800         1200         1600         3200         480           6400         8000         10000         15000         20000         30000         40000         50000         60000         800	
F1 F2 F3  Meat	
WEATHER DATA:  Array attached STAR File Name: HAR0214	
WBAN: 14751 Temperature: 12 C Header: 214 Lid Height: 900 meters Code: Rainfall: 105 cm/yr Set No:	
STACK Source: 1 2 3 4 5 6 Height (m):	
AREA Source Diameter (m):	
BUOYANT cal/sec	
X ENTERED Rise O O O O O O meters Pasquill Category: A B C D E F G	
Nuclide Class Diam Ci/vr:	

COMMENTS: Harrisburg 5-yr met qat: 10 miles NW of plant

File Name: TMI

Distances

							_				
_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N								21	237	3854	3651
NNW								54	109	1204	815
NM								44	137	98	56
WNW								25	144	364	204
W								40	460	51	475
WSW								32	342	168	1143
SW								94	150	261	808
SSW								94	199	840	306
S										177	1069
SSE								96	238	130	81
SE						•		7	112	_83	251
ESE								7	42	184	259
E								49	88	44	181
ENE								56	67	808	514
NE								47	157	302	478
NNE								61	84	189	536

### Distances

	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	465								
NNW	2163								
NW	1248								
WNW	359								
W	821						,		
WSW	<u>34</u> 0		1						
SW	300								
SSW	842								
S	1821								
SSE	54								
SE	489								
ESE	272								
E	649								
ENE	282								
NE	208								
NNE	416							<u> </u>	· · · · · · · · · · · · · · · · · · ·

. INSTALLATION: THREE MILE ISLAND I ....

LOCATION: 10 MI SE HARRISBURG, PA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: METROPOLITAN ED &

JERSEY CENTRAL P&L

DOCKET NO.: 50-289 LICENSED PWR (MWT): 2.57E+03
THERMAL PWR (MWH): 1.69E+07 NET ELECTRIC PWR (MWH): 5.47E+06
COMMERCIAL OPERATION: 09/02/74 INITIAL CRITICALITY: 06/05/74

COOLING WATER SOURCE: SUSQUEHANNA RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41 -	3,24E+Q0
CR-51	2.35E-07
CO-58	9.92E-07
KR-85	8.64E+00
KR-85M	1.34E+00
KR-87	5.57E-04
KR-88	1.10E-01
NB-95	1.38E-08
AC-110M	4.34E-08
I-131	1.26E-03
XE-131M	1.18E+01
I-132	3.31E-06
I-133	7.70E-04
XE-133	1.79E+03
XE-133M	1.44E+01
I-135	1.12E-03
XE-135	3.60E+01
XE-135M	1.38E-04

TOTAL AIRBORNE TRITIUM RELEASE 6.03E+00

INSTALLATION: THREE MILE ISLAND 2\*
LOCATION: 10 MI SE HARRISBURG, PA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: PWR LICENSEE: METROPOLITAN ED &

JERSEY CENTRAL P&L

DOCKET NO.: 50-320 LICENSED PWR (MWT): 2.77E+03
THERMAL PWR (MWH): 0.00E+00 NET ELECTRIC PWR (MWH): 0.00E+00
COMMERCIAL OPERATION: 12/30/78 INITIAL CRITICALITY: 03/28/78

COOLING WATER SOURCE: SUSQUEHANNA RIVER

### AIRBORNE EFFLUENTS

NUCLIDES RELEASED	ACTIVITY (CI)
KR-85	4.40E-01
SR-90	6.30E-08
CS-137	9.61E-06
UNIDENTIFIED	5.81E-05

TOTAL AIRBORNE TRITIUM RELEASE 1.14E+01

\*INCLUDING EPICOR II

### CAP-88 INFORMATION SHEET

Date: <u>April 1991</u>	- Source Cate	egory: <u>Reactor</u>	<u>'s</u>
Facility: Trojan Address: Prescott, OR			
Population Assessment Array attached	JCL File Name:	TROJAN	
Latitude: 46 • 2 • 26	Pop. File Name:	TROJAN	
Individual Assessment	JCL File Name:		<del></del>
Distances (meters)  200 300 400 500  6400 8000 10000 15000	600 800 120 20000 30000 400		3200 <u>4800</u> 0000 80000
Meat <u>.00</u> 8 <u>.</u> Milk <u>.000</u> <u>1</u>	2 F3 992 <u>.000</u> .00 <u>.000</u> 924 .000	Urban Rural	
WEATHER DATA:  Array attached	STAR File Name:	PDX0364	<del></del>
Header: 364 L	emperature: 12 °C id Height: 800 met lainfall: 95 cm		
STACK Source: 1 Height (m): 66 Diameter (m):	2 3	4 5	6
AREA Source Diameter (m*):	- <del></del> -		
BUOYANT cal/sec MOMENTUM m/s			
	B C D E	o o me	ters
Nuclide Class Diam Ci/yr:			

COMMENTS: Portland 5-yr met data ~40 miles S of plant

File Name: TROJAN

Distances

	200	300	400	500	600,	800	1200	1600	3200	4800	6400
N							174				
NNW							114		. 6	308	237
NW									6	_51	166
WNW											141
W									15	128	148
WSW			,					6	38	30	73
SW								15	44	113	166
SSW								6	24	73	141
S								30	103	118	24
SSE							0		97		- 6
SE									15	1517	1162
ESE									15	177	15_
E									_52	_111_	118
ENE									27	37	68
NE									52	81	127
NNE										304	324

Distances .

	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW	362								
NW	2133								
WNW	216								
W	118								
WSW	79								
SW	113								
SSW	223								
S	69								
SSE	15								
SE	329								
ESE	103								
E	96								
ENE	7								
NE	297								
NNE	490								

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INSTALLATION: TROJAN

LOCATION: 35 MI NW PORTLAND, OR

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: PORTLAND GEN ELEC LICENSED PWR (MWT): 3.41E+03
THERMAL PWR (MWH): 1.98E+07 NET ELECTRIC PWR (MWH): 6.34E+06
COMMERCIAL OPERATION: 05/20/76 INITIAL CRITICALITY: 12/15/75

COOLING WATER SOURCE: COLUMBIA RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	8.46E-01
CO-58	1.85E-04
CO-60	4.27E-06
KR-85	2.75E+00
KR-85M	3.55E-01
KR-87	9.72E-02
KR-88	1.17E-01
SR-89	1.74E-05
SR-90	7.29E-06
NB-95	3.15E-06
MO-99	1.08E-08
1-131	2.88E-03
XE-131M	2.51E+00
I-132	2.25E-07
I-133	1.02E-03
XE-133	3.84E+02
XE-133M	1.36E+00
CS-134	2.18E-08
I-135	7.01E-06
XE-135	6.45E+00
XE-135M	7.13E-01
CS-137	5.48E-08
XE-137	2.06E-01
XE-138	2.54E-01
BA-140	7.75E-09
ND-147	3.60E-05

TOTAL AIRBORNE TRITIUM RELEASE 6.44E+01

Date: Abril 1991 CAP-88 INFORMATION SHEET Source Category: Reactors
Facility: Turkey Point Units 3 & 4  Address: Florida City, FL
Population Assessment JCL File Name: TURKEY  Array attached Latitude: 25 26 05  Longitude: 80 19 53 Pop. File Name: TURKEY
Individual Assessment JCL File Name:
Distances (meters)  200 300 400 500 600 800 1200 1600 3200 4800 6400 8000 10000 15000 20000 30000 40000 50000 60000 80000
Food Fractions: F1 F2 F3  Meat .008 .992 .000
WEATHER DATA:  Array attached STAR File Name: TOL0990
WBAN: 12839 Temperature: 24 C Header: 979 Lid Height: 1400 meters Code: Rainfall: 146 cm/yr Set No:
STACK Source: 1 2 3 4 5 6  Height (m): 55
AREA Source Diameter (m):
BUOYANT cal/sec MOMENTUM m/s
X ENTERED Rise 0 0 0 0 0 0 0 meters Pasquill Category: A B C D E F G
Nuclide Class Diam Ci/yr:

COMMENTS Mismi met data 5-yr 35 miles N of plant

File Name: TURKEY

### Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N			•						3		
NNW											
NW											24
MNM											
W											
WSW											
SW			•								
SSW											
s											
SSE							`				
SE											
ESE											
E											
ENE											
NE											
NNE											

### Distances

	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW									
NW									
WNW									
W									
WSW									
SW									
SSW									
S									
SSE									
SE									
ESE									
E									
ENE									
NE									
NNE									

INSTALLATION: TURKEY POINT 3

LOCATION: 10 MI E FLORIDA CITY, FL

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 3 TYPE: PWR LICENSEE: FLORIDA P&L

DOCKET NO.: 50-250 LICENSED PWR (MWT): 2.20E+03
THERMAL PWR (MWH): 1.14E+07 NET ELECTRIC PWR (MWH): 3.45E+06
COMMERCIAL OPERATION: 12/14/72 INITIAL CRITICALITY: 10/20/72

COOLING WATER SOURCE: CLOSED CYCLE CANAL

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	8.44E-01
CR-51	1.24E-05
MN-54	2.62E-06
CO-58	1.99E-05
CO-60	1.70E-05
BR-82	6.22E-04
KR-85	1.38E+00
KR-85M	5.98E-01
KR-87	8.26E-03
KR-88	4.00E-02
SR-89	2.13E-06
I-131	3.90E-03
XE-131M	1.56E+01
I-133	4.23E-03
XE-133	1.20E+03
XE-133M	6.94E+00
CS-134	2.06E-05
I-135	1.97E-03
XE-135	1.87E+01
CS-136	6.06E-06
CS-137	,6.13E-05
BA-140	2.93E-05
LA-140	1.90E-06
CE-144	1.20E-05

TOTAL AIRBORNE TRITIUM RELEASE 2.01E+02

INSTALLATION: TURKEY POINT 4 LOCATION: 10 MI E FLORIDA CITY, FL

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 4 TYPE: PWR LICENSEE: FLORIDA P&L

DOCKET NO.: 50-251 LICENSED PWR (MWT): 2.20E+03
THERMAL PWR (MWH): 1.06E+07 NET ELECTRIC PWR (MWH): 3.26E+06
COMMERCIAL OPERATION: 09/07/73 INITIAL CRITICALITY: 06/11/73

COOLING WATER SOURCE: CLOSED CYCLE CANAL

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	2.91E+01
CR-51	1.24E-05
MN-54	2 5 <b>2E-06</b>
CO-58	1.99E-05
CO-60	1.68E-05
BR-82	6.22E-04
KR-85	1.51E+00
KR-85M	6.36E-01
KR-87	7.30E-03
KR-88	7.68E-02
SR-89	2.13E-06
I-131	3.85E-03
XE-131M	1.38E+01
I-133	4.22E-03
XE-133	1.23E+03
XE-133M	7.51E+00
CS-134	2.06E-05
I-135	1.97E-03
XE-135	1.94E+01
CS-136	6.06E-06
CS-137	6.11E-05
BA-140	2.93E-05
LA-140	1.90E-06
CE-144	1.20E-05

TOTAL AIRBORNE TRITIUM RELEASE 2.01E+02

Date: April 1991 Source Category: Reactors
Facility: Virgil C. Summer  Address: Jenkinsvill, SC
Population Assessment JCL File Name:  Array attached Latitude: 34 17 44 " Longitude: 81 19 13 Pop. File Name:
Individual Assessment
Distances (meters):         200       300       400       500       600       800       1,200       1,600       3,200       4,800         6,400       8,000       10,000       15,000       200       30,000       40,000       50,000       60,000       80.000
Food Fractions:  F1 F2 F3  Meat  Milk Urban Rural
Veg
WBAN: 18883 Temperature: 19 °C  Header: 1371 Lid Height: 900 meters  Code: Rainfall: 125 cm/yr  Set No:
STACK Source: 1 2 3 4 5 6  Height (m) 27
AREA Source Diameter (m):
BUOYANT cal/sec
X ENTERED Rise 0 0 0 0 0 0 meters Pasquill Category: A B C D E F G
Nuclide Class Diam Ci/vr:

COMMENTS:

Columbia 20 year met data, approximately 25 miles SE of plant.

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File Name: SUMMER

### Distances

N NNW WNW WSW SW								<u>57</u> 11
WNW W W								
WNW W WSW								11
WSW							7	7
WSW			 					
							11	14
SW	·							14
								50
SSW					 			32
S			 		 		4	• 64
SSE							4	14
SE	•		 	0		4	28	18
ESE					 	36	18	14
E		_			4	14	14	21
ENE	•		 		 4	43	43	21
NE						39	64	14
NNE			 					

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	00008
N	78								
NNW	46								
NM	28								<del></del>
WNW	25						,		
W	36								
WSW	32								
SW	43								
SSW	28								
S	65								
SSE									
SE									
ESE	39								
E	11								
ENE	21	· · · · · · · · · · · · · · · · · · ·							
NE	7								
NNE	14								

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INSTALLATION: SUMMER

LOCATION. 26 MI NW OF COLUMBIA, SC

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER 1 TYPE PWR

UNIT NUMBER 1 TYPE PWR

DOCKET NO.: 50-395

THERMAL POWER (MWH). 1.61E+07

COMMERCIAL OPERATION. 01/01/84

LICENSEE: SOUTH CAROLINA ELECTRIC & GAS

LICENSED POWER (MWT): 2.77E+03

NET ELECTRIC POWER (MWH): 5.05E+06

INITIAL CRITICALITY: 10/22/82

COOLING WATER SOURCE: MONTICELLO RESERVOIR

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	3.73E-01
CO-58	1 10E-05
KR-85	4 09E+00
KR-85M	2.03E+00
KR-87	5.25E-03
KR-88	2.91E+00
RB-88	8.44E-03
1-131	2.28E-03
XE-131M	1.52E+00
I-132	2.30E-04
· I-133	1.13E-03
XE-133	2.97E+02
XE-133M	1 06E+00
CS-134	3.46E-06
1-134	2.10E-06
1-135	1.05E-04
XE-135	2.27E+01
CS-136	6.90E-06
CS-137	2.25E-05
CS-138	2.75E-04

#### TOTAL AIRBORNE TRITIUM RELEASE 1 51E+00

N/A=NOT APPLICABLE

N/D-NOT DETECTED

N/R=NOT REPORTED

**.CAP-88 INFORMATION SHEET** Date: \_ spril 1991 Source Category: Reactors Waterford Unit 3 Facility: Taft, LA Address: WATERFOR Population Assessment JCL File Name: Array attached 29 • 59 43 • Latitude: 90 • 25 22 Pop. File Name: WATERFOR Longitude: Individual Assessment JCL File Name: Distances (meters) 400 500 600 800 1200 1600 3200 4800 200 300 10000 15000 20000 30000 40000 8000 6400° 50000 60000 80000 Food Fractions: F1 F2 F3 Urban Milk .399 .601 .000 Veg .700 .300 .000 Rural WEATHER DATA: STAR File Name: \_\_\_\_NBG1379 Array attached 12958 Temperature: 20 WBAN: t 1379 900 Lid Peight: Header: meters Code: Rainfail: cm/yr Set No: STACK Source: 2 3 5 6 Height (m): Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec \_\_\_\_\_ \_\_\_\_ MOMENTUM m/s \_\_\_\_\_ 0 0 0 0 0 B C D E F X ENTERED Rise Pasquill Category:

COMMENTS: New Orleans 5-yr 30 milesE of plant

Class Diam Ci/vr:

<u>Nuclide</u>

File Name: W terfor 3

	·				Di	stance	5				
	200	300	-400	500	600	800	1200	1600	3200	4800	6400
N								88	29	4	294
NNW								50	462	260	416
NW								3			216
WW								11	472	65	
W								76	209		
WSW						•		18	29		
SH											
SSW											
S											
SSE											
SE							`				
ESE											70∠
E								47	76	138	2074
ENE								3		1110	4134
NE								3			<u>.19</u> 5
NNE								118			8

Distances

					v	ائا ہوں در			
_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	269								
MNM	1693	_							
NW	364								
MNM	24				-				
W									
WSW		_							
. SW						· · · · · · · · · · · · · · · · · · ·			
SSW		_							
S									
SSE									
SE	3								
	644								
ET	-755								
ENE	39								
NE									
NNE									

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INSTALLATION: WATERFORD

LOCATION: 20 MI W NEW ORLEANS, LA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 3 TYPE: PWR LICENSEE: LOUISIANA P&L

DOCKET NO.: 50-382

THERMAL PWR (MWH): 1.87E+07

COMMERCIAL OPERATION: 09/24/85

LICENSED PWR (MWT): 3.39E+03

NET ELECTRIC PWR (MWH): 6.00E+06

INITIAL CRITICALITY: 03/04/85

COOLING WATER SOURCE: MISSISSIPPI RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
ALPHA	1.88E-04
AR-41	5.26E-01
CR-51	6.56E-05
MN-54	8.88E-06
CO-57	1.11E-07
CO-58	1.39E-04
CO-60	3.46E-05
KR-85	1.29E+01
KR-85M	3.60E-01
KR-88	1.57E-01
NB-95	<b>5.30</b> E-05
2R-95	2.34E-05
RU-103	1.45E-07
RU-106	1.99E-06
I-131	8.69E-04
XE-131M	1.75E+01
I-133	5.51E-06
XE-133	5.08E+03
XE-133M	5.12E+00
CS-134	2.95E-07
XE-135	1.88E+02
CS-137	4.13E-06
HG-203	4.03E-07

TOTAL AIRBORNE TRITIUM RELEASE 2.46E+02

N/A=NOT APPLICABLE

N/D=NOT DETECTED

N/R=NOT REPORTED

Date: A			2-88 INFO	KMAHON	Source	Category:	Rescto	rs	
acility:			k Unit	1					
		attache	d			WOI	P	<del></del>	
ī	atitude: .ongitude:	84	43 23	Pop. F	ile Name	WOL	<u> </u>	···	
l	ndividual A	emezeez	nt	JCL File	Name:		· · · · · · · · · · · · · · · · · · ·		
200	es (meters) 300 8000	400	_500 15: 00	_600 20000	800 30000	1200 40000		3200 A	
Food F	ractions:	Meat Milk	F1 F2 -442 -51 -399 -60 -700 -30	<u> </u>	_	Urbar Rural	n	,	
VEATH	IER DATA: Array	attache				.me: <u>ஈ</u> റ	P534		
WBAN: Header: Code: Set No:			Lid		13 _ 900 _ 73	meters			
	STACK Sou Height (m): Diameter (m		1 64	2	3	4	5	6	-
	AREA Sour Diameter (n								-
E	BUOYANT ( MOMENTU	cal/sec M m/s							- -
X	ENTERED I Pasquill Car	Rise tegory:	0 0 A B	<u>o</u>	<u>0</u> –	0 0 E F	G me	eters	
Nuclide	Class	<u>Diam</u>	Ci/yr:				<u> </u>		_
									- -
СОММ	ENTS: ¥	erafy ]	LAT & LO	NG M	et d ta	Topek'	10-yr 6	O miles	N of plan

File Name: Wolf

		Distances									
	200	300	400	500	600	800	1200	1600	3200	4800	6400
N									2	2	6
NNW									1		
NW	•								1	593	532
WIM										2	1474
M										8	1
MSH									1	7_	23 655
SH	·-			<del>-</del>						3_	655
SSW					,					1	
S											8
SSE									1		
SE						_			2	4	- 4
ESE						-			6	4	2
E	<u>-</u>								2	1	
ENE										4	2
NE									1	3	8
NNE									1	1	4
				•							<del></del>

Distances 8000 10000 15000 20000 30000 40000 50000 60000 80000 N MNM 4 6 NH MNM 475 W WSW 11 SW1456 SSW \$ 6 SSE 6 SE ESE 11 E 1/2 ENE NE NNE 13

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INSTALLATION: WOLF CREEK

LOCATION: 3.5 MI NE BURLINGTON, KANSAS

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

LICENSEE: KANSAS G&E UNIT NUMBER: 1 TYPE: PWR

DOCKET NO.: 50-482 LICENSED PWR (MWT): 3.41E+03

THERMAL PWR (MWH): 2.01E+07

COMMERCIAL OPERATION: 09/93/85

LICENSED PWR (MWT): 3.41E+03

NET ELECTRIC PWR (MWH): 6.66E+06

INITIAL CRITICALITY: 05/22/85

COOLING WATER SOURCE: COOLING LAKE

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
д т ФНД	8.05E-08
AR-41	9.72E-01
CO-58	1.09E-05
CO-60	1.61E-05
KR-85	1.17E+00
KR-85M	7.66E-01
KR-87	2.82E-04
KR-88	1.10E+00
I-131	5.66E-05
XE-131M	4.09E+00
XE-133	7.57E+02
XE-133M	7.66E+00
XE-135	1.88E+01

### TOTAL AIRBORNE TRITIUM RELEASE 1.46E+02

N/A=NOT APPLICABLE

N/D=NOT DETECTED

N/R=NOT REPORTED

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### CAP-88 INFORMATION SHEET

Date:	<u>April 1991</u>	Source Cate	egory: <u>Reactors</u>	
Facility Addres				
	Population Assessment JC  Array attached	L File Name:	YANKEE	
	Latitude: 42 43 41	op. File Name:	YANKEE	
	Individual Assessment JC	L File Name:		
Distant 200 6400			200 1600 32 00 50000 600	00 4800 00 80000
Food (	Mik .000 1.00 .	3 000 000 000	Urban Rural	
WEAT	HER DATA: Array attached	STAR File Name:	CEFO182	
WBAN Heade Code: Set No	er: 182 Lid Heig CEF Rainfall:	thi: 850 met		
	STACK Source: 1 Height (m): 46 Diameter (m):	2 3	4 5	6 .
	AREA Source Diameter (m*):			
_	BUOYANT cal/sec		:	
<u>x</u> _	ENTERED Rise 0 0 Pasquill Category: A B	0 0 0 C D E	F G meter	rs
Nuclid				
COMI	MENTS: Chicopee Falls/West	over 5-yr met	data ~35 miles	SE of plant

File Name: YANKEE

Distances

							-				
_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N										22	18
NNW								5		47	311
NW				3					5	23	23
MNM									19		10
W									19	33	9
wsw								65	28	179	1
SW								28			17_
SSW											6_
S									19	21	13_
SSE		-							21	77	· 2
SE		_							10_	59	
ESE								<del> </del>		13	10_
E									4	13	42
ENE									6	2	8
NE										4	12
NNE										15	31

Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	17								
NNW	20								
NW	17								
WNW	20								
W									
WSW	6								
SW	37								
SSW	3								
S	24								
SSE	30								
SE	10								
ESE	39								
E	21								
ENE	27			•					
NE	108								
NNE	169								

INSTALLATION: YANKEE ROWE

LOCATION: 20 MI NW GREENFIELD, MA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: YANKEE ATOMIC ELECTRIC DOCKET NO.: 50-29 LICENSED PWR (MWT): 6.00E+02 THERMAL PWR (MWH): 4.03E+06 NET ELECTRIC PWR (MWH): 1.12E+06 INITIAL CRITICALITY: 08/19/60

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COOLING WATER SOURCE: DEERFIELD RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI
C-14	6.41E-01
AR-37	7.11E-01
AR-41	<8.64E-01
CR-51	<1.24E-05
MN-54	<2.26E-06
CO-58	<2.18E-06
_FE-59	<4.48E-06
CO-60	6.98E-06
ZN-65	<4.84E-06
KR-85	4.10E+00
KR-85M	1.94E+00
KR-87	<1.69E+00
KR-88	<3.53E+00
SR-89	<9.84E-07
SR-90	<4.32E-07
ZR-NB-95	<3.51E-06
MO-99	<1.37E-05
RU-103	<1.67E-06
AG-110M	<1.92E-06
SB-124	<1.99E-06
1-131	5.04E-05
XE-131M	1.10E+00
I-133	1.61E-05
XE-133	1.02E+02
XE-133M	2.07E+00
CS-134	<1.78E-06
I-135	<1.49E-06
XE-135	3.89E+01
XE-135M	<4.75E+01
CS-137	1.50E-06
XE-138	<8.20E-01
BA-LA-140	<6.07E-06
CE-141	<1.69E-06
CE-144	<7.33E-06

TOTAL AIRBORNE TRITIUM RELEASE 4.58E+00

**CAP-88 INFORMATION SHEET** 

Date: _Arril 1991	Source Cate	gory: Resct	ors					
Facility: Zion 1 & 2 Address: Waukegon, IL								
Population Assessment JCL File Array attached	Name:	ZION	-					
Latitude: 42 26 44 Pop. F	ile Name:	ZION						
Individual Assessment								
Distances (meters)       200     300     400     500     600       6400     8000     10000     15000     20000		200 1600 5000 50000	3200 4800 60000 80000					
Food Fractions: F1 F2 F3		l labara						
Meat <u>.008 .932 .000</u> Milk <u>.000 1.00 .000</u> Veg <u>.076 .924 .000</u>		Urban Rural						
WEATHER DATA:	R File Name:	ольо452						
WBAN: 94846 Temperature: Header: 452 Lid Height: Rainfall: Set No:	10 °C 1950 meter 25 cm/	_	`					
STACK Source: 1 2 Height (m): 55 Diameter (m):	3	4 5						
AREA Source Diameter (m ):	1							
BUOYANT cal/sec MOMENTUM m/s								
X ENTERED Rise 0 0 0 Pasquill Category: A B C	0 0 E	0 0 f	meters					
Nuclide Class Diam Ci/vr:								
COMMENTS: Chic-go/Ohere 5-yr 35	s of pla	.nt						

### File Name: ZION

### Distances - ---

	200	300	400	500	600	800	1200	1600	3200	4800	6400
<u> </u>										1408	497
NNW									1902	1408	496
NW									1902	1409	495
WNW							0 -	96	1902	1408	496
<u> w</u>								96	1902	1408	495
WSW								97	1902	1409	496
SW									1902	1408	495
SSW									1902	1408	496
S									1903	1409	495
SSE											
SE											
ESE				-							
E											
ENE											
NE											
NNE											

### Distances

_	8000	10000	15000	20000	30000	40000	<b>5000</b> 0	60000	80000
N	733								
NNW	733								
NW	733								
WNW	733								
W	734								
WSW	733								
SW	733								
SSW	733								
S	734								
SSE									
SE									
ESE									
E									
ENE									
NE									
NNE									

INSTALLATION: ZION

LOCATION: 6 MI N WAUKEGAN, IL

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: PWR LICENSEE: COMMONWEALTH EDISON DOCKET NO.: 50-295 LICENSED PWR (MWT): 3.25E+03
THERMAL PWR (MWH): 1.97E+07 NET ELECTRIC PWR (MWH): 6.34E+06

COMMERCIAL OPERATION: 12/31/73 INITIAL CRITICALITY: 06/19/73

COOLING WATER SOURCE: LAKE MICHIGAN

UNIT NUMBER: 2 TYPE: PWR LICENSEE: COMMONWEALTH EDISON DOCKET NO.: 50-304 LICENSED PWR (MWT): 3.25E+03 THERMAL PWR (MWH): 2.08E+07 NET ELECTRIC PWR (MWH): 6.65E+06 COMMERCIAL OPERATION: 09/17/74 INITIAL CRITICALITY: 12/24/73

COOLING WATER SOURCE: LAKE MICHIGAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
NA-24	1.03E-05
AR-41	1.04E-01
MN-54	1.90E-08
MN~56	1.30E-07
CO-58	<b>5.77E-0</b> 5
CO-60	1.51E-04
BR-82	3.01E-05
BR-84	6.20E-07
KR-85	3.40E+00
KR-85M	1.14E-02
KR-88 ·	1.31E-02
RB-88	6.43E-03
SR-89	3.20E-07
NB-95	<b>8.20E-</b> 09
ZR-95	2.50E-09
MO-99	4.57E-07
_TC-99M	9.30E-08
AG-110M	2.10E-07
SN-113	1.30E-10
I-130	4.60E-08
I-131	1.15E-03
XE-131M	6.44E-01
I-132	1.65E-03
I-133	4.09E-04
XE-133	1.39E+03
XE-133M	3.56E-01
CS-134	1.94E-05
I-134	2.63E-04
I-135	1.82E-04
XE-135	5.53E+01
CS-136	1.09E-06
CS-137	2.26E-05
CS-138	3.94E-04
BA-139	. 5.90E-07

### INSTALLATION: ZION\_\_\_\_

BA-140		1.64E-07
LA-140		2.02E-08
BA-142	-	8.10E-04
W-187		3.66E-07

### TOTAL AIRBORNE TRITIUM RELEASE 3.69E+02

Boiling Water Reactors (BWRs)

### **CAP-88 INFORMATION SHEET**

Date: April	1991	Source Car	eBold: 'Hé-ctuis"	to the same
Facility: .	Big Rock Point Charleyoix. MI			<del></del>
•		ICI Ele Neme		-
	tion Assessment Array attached	JCL File Name:	RICROCK	<del></del>
Latitude Longitu	de: 45 21 33 de: 85 11 40 de:	Pop. File Name:	BIGROCK	
Individu	ual Assessment	JCL File Name:		<del></del> _
Distances (me 200 300 6400 800	400 500	600 800 12 20000 30000 40	200 1600 320 0000 50000 600	
Food Fraction		F3		
	Meat OOS 99	0 .000	Urban Rural	
WEATHER DA	Veg <u>.076 <del>▼</del>9</u> <u>4TA</u> :	24 <u>.0</u> 00		
` '	Array attached	STAR File Name:	TVC0844	<del></del>
WBAN: Header: Code: Set No:		perature: 40 °C Height: 800 me fall: 85 cm	ters /yr	•
STACK Height Diamete		2 3	4 5	6 .
	Source er (m²):			
BUOYA	NT cal/sec			
X ENTER	ED Rise 0 0  I Category: A B	0 0 0 C D E	F G meter	rs
Nuclide	Class Diam Ci/vr:			
•				
COMMENTS:	Met data Traverse	City 20 miles	SW of plant	

### Distances

	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											
NNW											
NW							,				
MNM											
W											
WSW										123	279
SW	-								_11	113	222
SSW									. 10	114	222
S									9	21_	25
SSE									9	21_	24
SE									16	34	23
ESE									16	34	23
E									4	18	17
ENE										18	18
NE											
NNE											

### Distances

_	8000.	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW									
NW									
WNW									
W									
WSW									
SW	355	5							
SSW	356	; ;							
S	_28	}							
SSE	2	>							
SE									
ESE									
E									
ENE									
NE									
NNE									

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INSTALLATION: BIG ROCK-POINT

LOCATION: 4 MI NE CHARLEVOIX, MI

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: CONSUMERS PWR

DOCKET NO.: 50-155

THERMAL PWR (MWH): 1.26E+06

COMMERCIAL OPERATION: 03/29/63

LICENSED PWR (MWT): 2.40E+02

NET ELECTRIC PWR (MWH): 3.84E+05

INITIAL CRITICALITY: 09/27/62

COOLING WATER SOURCE: LAKE MICHIGAN

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
N-13	1.47E+03
NA-24	1.97E-04
CR-51	5.12E-04
MN-54	1.61E-04
CO-58	1.91E-06
FE-59	7.36E-06
CO-60	2.42E-04
ZN-65	9.26E-05
BR-82	3.11E-03
KR-85M	1.13E+02
KR-87	5.62E+02
KR-88	3.54E+02
SR-89	2.03E-04
SR-90	3.37E-06
SR-91	3.72E-03
NB-95	<b>6.22E-0</b> 6
MO-99	1.04E-04
AG-110M	6.92E-04
I-131	2.15E-03
I-133	2.05E-02
XE-133	4.26E+01
CS-134	8.11E-05
-I-135	2.51E-02
XE-135	5.33E+02
XE-135M	9.16E+02
CS-137	2.96E-04
XE-138	3.78E+03
BA-139	9.76E-04
BA-140	5.30E-04
LA-140	1.55E-03
HG-203	2.81E-06
NP-239	<b>5.63E-0</b> 5
UNIDENTIFIED	1.77E-04

TOTAL AIRBORNE TRITIUM RELEASE 5.30E+00

Date: May 1991	CAP-88 INFOR	RMATION SHEE Source	T a Category: _	Reactors	
Facility:Address:	Browns Ferry NPS	, Units 1, 2, a	and 3		<del></del>
Array	Assessment attached	JCL File Name		WNS	_
*		' Pop. File Nar	ne: <u>BRC</u>	WNS	_
Individual A	ssessment	JCL File Name	:		_
Distances (meters)  200 300  6400 8000	400 500	600 800 20,000 30,00		1600 <u>3200</u> 50,000 60,000	4800 80,000
Food Fractions:	F1 F2 Meat <u>.008</u> <u>.992</u> Milk <u>.000</u> 1.000	2 -000	Urban Rural		
Veg076924000  WEATHER DATA:  Array attached STAR File Name:HSV0544					
1100001	44 Lid	perature:			
STACK Sou Height (m): Diameter (n	183	2 3	4	5 =====================================	6 .
AREA Sou Diameter (n	- **				
BUOYANT MOMENTU	cai/sec M m/s				
X ENTERED Pasquill Ca		$-\frac{\mathbf{C}}{0}$	0 0 E F	O meters	
Nuclide Class	S Diam Ci/yr:				
COMMENTS: H	duntsville Met Data	a approximatel	y 25 miles NE	of plant (5-y	ear)

C-165

File Name: BROWNS

					Di	stance	s				
	200	300	400	500	600	800	1200	1600	3200	4800	6400
₩									10	45	30
NNW								5		30	20
MM.											
WNW											25
W										25	5
MSW										35	15
SW										25	40
SSW										60	75
S										20	35
SSE											
SE											
ESE										10	
Ε										65	25
ENE									30	105	85
NE									5	<b>3</b> 5	55
NNE									5	15	55

Distances

_	<b>6030</b>	10000	15000	20000	30000	40900	50000	60000	80000
พ	70								
NNW	15								
NM	5								
WNW	55								
W	30								
WSW	140								
SW	95								
SSW	170								
S	95								
SSE	50								
SE	20			•					
ESE	`								
E	85								
ENE	150								
NE	70								
NNE	45								

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INSTALLATION: BROWNS FERRY
LOCATION: 10 MI NW DECATUR, AL

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1983 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: TENNESSEE VALLEY AUTHORITY

DOCKET NO.: 50-259

LICENSED POWER (MWT): 3.29E+03

THERMAL POWER (MWH): 0.00E+00

COMMERCIAL OPERATION: 08/01/74

LICENSED POWER (MWT): 3.29E+03

NET ELECTRIC POWER (MWH): 0.00E+00

INITIAL CRITICALITY: 08/17/73

COOLING WATER SOURCE: TENNESSEE RIVER

UNIT NUMBER. 2 TYPE: BWR LICENSEE: TENNESSEE VALLEY AUTHORITY

DOCKET NO.: 50-260

THERMAL POWER (MWH): 0.00E+00

COMMERCIAL OPERATION: 03/01/75

LICENSED POWER (MWT): 3.29E+03

NET ELECTRIC POWER (MWH): 0 00E+00

INITIAL CRITICALITY: 07/20/74

COOLING WATER SOURCE: TENNESSEE RIVER

UNIT NUMBER 3 TYPE: BWR LICENSEE TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-296 LICENSED POWER (MWT): 3.29E+03
THERMAL POWER (MWH) · 0.00E+00
COMMERCIAL OPERATION: 03/01/77 INITIAL CRITICALITY · 08/08/76

COOLING WATER SOURCE: TENNESSEE RIVER

#### AIRBORNE EFFLUENTS

NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	1 68E+03
MN-54	2.99E-03
CO-58	2.52E-03
FE-59	6.43E-03
CO-60	1.15E-02
ZN-65	1.53E-02
KR-85	5 04E+03
KR-85M	5.56E+04
KR-87	2 85E+04
KR-88	6 07E+04
SR-89	3.72E-03
SR-90	3 68E-04
NB-95	2.00E-02
ZR-95	1.25E-02
1-131	1.96E-01
1-133	7.08E-02
XE-133	2.66E+05
CS-134	4.33E-03
1-135	3 99E-02
XE-135	6.73E+03
XE-135M	3.17E+04
CS-137	4.80E-03
XE-138	2.41E+04
BA-140	7.39E-03

TOTAL AIRBORNE TRITIUM RELEASE 4.09E+01

N/A=NOT APPLICABLE N/D=NOT DETECTED N/R=NOT REPORTED

LA-140

2.29E-02

Date: <u>April 1991</u>	AP-88_INEORMATIO	N SHEET Source Category	Reactors			
Facility: Brunswick Southport	Steam Electri	c Plant, 1 & 2		<u> </u>		
Population Assessment JCL File Name: BRUNSWIC Array attached						
Latitude: 33 Longitude: 78	57 29 Pop.	File Name: BR	UNSWIC	_		
Individual Assessm	ent JCL Fil	e Name:		_		
Distances (meters)  200 300 400 6400 8000 10000	500 600 15000 20000	800, 1200 30000 40000	1600 3200 50000 60000	4800 80000		
Food Fractions:  Meat  Milk  Veg	.000 1.00 .000	Rura				
WEATHER DATA:  Array attach	ed ST/	AR File Name:	LM0104	_		
WBAN: 13748 Header: 104 Code: 16M Set No:	Tempe:ature Lid Height: Rainfall:	15 C 700 meters 106 cm/yr				
STACK Source: Height (m): Diameter (m):	100 2	3 4	5 6	_		
AREA Source Diameter (m.):						
BUOYANT cal/sec MOMENTUM m/s						
ENTERED Rise Pasquill Category:	A B C	D E F	O meters			
Nuclide Class Diam				<del></del> -		

COMMENTS: 5-yr met data from Wilmington 40 miles N of plant

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File	Name:	BRUNSWIC
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200   300   400   500   600   800   1200   1500   3200   4800   6400     Now						Di:	stance	s ,	•			
NNW 9 31 49 NW 11 19 31 31 WNW 9 19 W 19 43 WSW 28 175 181 SW 11 19 0 100 SSW 0 169 1340 496 S 292 1650 SSE 448 9 19 SE 11 ESE 9 13 E NE	_	200	300	400	500	600	800	1200	1600	3200	4800	6400
NW     11     19     31     31       WNW     9     19       WSW     28     175     181       SW     11     19     0     100       SSW     0     169     1340     496       S     292     1650       SSE     448     9     19       SE     9     13       ESE     9     13       ENE     12				-					11			
WNW     9     19       WSW     19     43       WSW     28     175     181       SW     11     19     0     100       SSW     0     169     1340     496       S     292     1650       SSE     448     9     19       SE     11       ESE     9     13       E     13       ENE       NE	NNW								9		31	49
WNW 9 19 WSW 28 175 181 SW 11 19 0 100 SSW 0 169 1340 496 S 292 1650 SSE 448 9 19 SE 11 ESE 9 13 ENE	NW								11	19	31	31
WSW 28 175 181 SW 11 19 0 100 SSW 0 169 1340 496 S 292 1650 SSE 448 9 19 SE 11 ESE 9 13 ENE	MNM								9	19		
SW     11     19     0     100       SSW     0     169     1340     496       S     292     1650       SSE     448     9     19       SE     11       ESE     9     13       E     13       ENE       NE	W									19	43	
SSW     0     169     1340     496       S     292     1650       SSE     448     9     19       SE     11       ESE     9     13       E     13       ENE       NE	WSW									28	175	181
S     292     1650       SSE     448     9     19       SE     11     ESE     9     13       E     13     ENE     NE	. SW					-			11	19	0	100
SSE     448     9     19       SE     11       ESE     9     13       E     13       ENE       NE	SSW					0		<u> </u>		169	1340	496
SE     11       ESE     9     13       E     13       ENE     NE	S						_			292	1650	
ESE 9 13  E 13  ENE NE	SSE					448		_	9	19		*
E 13 ENE NE	SE								11			
ENE NE	ESE								9	13		
NE	E			_						13		
NNE 9	NE											
	NNE								9			

### Distances

	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	45								<del></del>
MNM	194			<del></del>					
NW	30			-					
MNW									
W									
MSM	45								
SW	298				·				
SSW	185								
S	0							<del></del>	
SSE									
SE		_		<u> </u>					
ESE									
E	267								
ENE									
NE	18				<del></del>				
NNE	С								

INSTALLATION: BRUNSWICK

LOCATION: -20 MI S WILMINGTON, NC

#### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: CAROLINA PWR & LIGHT **DOCKET NO.: 50-325** LICENSED PWR (MWT): 2.44E+03 THERMAL PWR (MWH): 1.43E+07

THERMAL PWR (MWH): 1.43E+07

NET ELECTRIC PWR (MWH): 4.45E+06 COMMERCIAL OPERATION: 03/18/77 INITIAL CRITICALITY: 10/08/76

COOLING WATER SOURCE: CAPE FEAR RIVER

UNIT NUMBER: 2 TYPE: BWR LICENSEE: CAROLINA PWR & LIGHT DOCKET NO.: 50-324 LICENSED PWR (MWT): 2.44E+03 THERMAL PWR (MWH): 1.24E+07

COMMERCIAL OPERATION: 11/03/75

EICENSED PWR (MWT): 2.44E+03

NET ELECTRIC PWR (MWH): 3.92E+06

INITIAL CRITICALITY: 03/20/75

COOLING WATER SOURCE: CAPE FEAR RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	2.17E+Ò1
CR-51	1.30E-01
MN-54	6.50E-03
CO-57	2.66E-06
CO-58	3.99E-03
FE-59	1.39E-03
CO-60	7.73E-03
ZN-65	5.73E-05
KR-85M	7.08E+01
KR-87	1.07E+01
KR-88	6.11E+01
SR-89	9.91E-04
SR-90	9.15E-06
NB-95	1.77E-05
RU-103	9.27E-06
I-131	2.27E-02
I-132	3.80E-02
I-133	4.35E-02
XE-133	4.35E+02
XE-133M	1.13E+00
CS-134	1.41E-04
I-134	2.27E-02
I-135	4.57E-02
XE-135	6.10E+02
XE-135M	8.45E+01
CS-136	3.73E-06
CS-137	4.41E-04
XE-137	2.30E+02
XE-138	5.05E+01 6.14E-04
BA-140	1.91E-03
BA-LA-140 LA-140	4.20E-04
LA-140 CE-141	4.25E-04 4.25E-08
CE-141 CE-144	2.06E-05
CD-144	2.002-03

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INSTALLATION: BRUNSWICK

TOTAL AIRBORNE TRITIUM RELEASE 5.55E+00

CAP-88 II	NFORMATION SHEET Source Cate	egory: <u>Re.ctor</u>	S . m. s. mediane
Facility: Clinton Address: Clinton,	Forer Station		
Population Assessment Array attached Latitude: 40 9 9	JCL File Name:	CLINTON	
Longitude: 28 57	Pop. File Name:	CLINTON	<del></del>
Individual Assessment	JCL File Name:		<del></del>
Distances (meters)       200     300     400     50       6400     8000     10000     150	00 600 800 12 000 20000 30000 40	704 <del>2000</del>	000 4800 0000 80000
Meat .008 Milk .000 1	F2 F3 .992 .000 .00 .000	Urban Rural	
WEATHER DATA:  Array attached	STAR File Name:	R# NO234	<del></del>
	Temperature: 10 °C Lid Height: 550 met Rainfall: 89 cm,		
STACK Source: 1 Height (m): 63 Diameter (m):	2 3 	4 5	6 .
AREA Source Diameter (m):			
BUOYANT cal/sec MOMENTUM m/s			
ENTERED Rise Pasquill Category:  A		F G met	ere
Nuclide Class Diam Ci/yr			

COMMENTS: Arntoul/Chanute AFB met data 5-yr 40 miles ENF of plant

## File Name: - CLINTON

### Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N						0		6		5	20
NNW								0		8	12
MM										10	5
WNW								0	10	10	5
W						0				12	12
WSW											12
SW								6	. 0		14
SSW											155
S			0_				0	i		11	8
SSE						0				8	3
SE						0				3	9
ESE											6
E								3	5	6	2
ENE		-			0			5		129	6
NE									5		3
NNE								3	8	5	17

### Distances

8000	<u> </u>	10000	15000	20000	30000	40000	50000	60000	80000
17	7_								
6	6							•	
15	5								
23	3								
23	3								
28	8								
88	8								
28	8								
19	9								
6	6								
11	1								
9	9								
9	9								
10	0								
9	9							-	
	1 2 2 2 1 2 1	17 6 15 23 12 23 28 8 19 28 19 6 11	17 6 15 23 12 23 28 8 19 28 19 6 11	17 6 15 23 12 23 28 8 19 28 19 6 11	17 6 15 23 12 23 28 8 19 28 19 28	17 6 15 23 12 23 28 8 19 28 19 28	17 6 15 23 12 23 28 8 19 28 19 6 11	17 6 15 23 12 23 28 8 19 28 19 6 11	17 6 15 23 12 23 28 8 19 28 19 6 11

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INSTALLATION: CLINTON

LOCATION: 6 MI'E CLINTON, IL

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: ILLINOIS PWR

DOCKET NO.: 50-461 LICENSED PWR (MWT): 2.89E+03
THERMAL PWR (MWH): 1.86E+07 NET ELECTRIC PWR (MWH): 5.86E+06

COMMERCIAL OPERATION: 11/24/87 INITIAL CRITICALITY: 02/27/87

COOLING WATER SOURCE: SALT CREEK

AIRBORNE	<b>EFFLUENTS</b>	
NUCLIDES	RELEASED	ACTIVITY (CI)
NA-24		2.74E-03
CR-51		5.36E-02
MN-54		2.08E-03
CO-58		1.41E-03
FE-59		2.45E-04
CO-60		1.77E-03
AS-76		3.27E-05
SR-89		5.81E-05
SR-90		8.44E-07
Y-91M		2.61E-05
TC-99M		4.65E-04
I-131		2.45E-04
I-133		4.77E-04
I-134		7.70E-05
XE-135		4.34E+00
CS-138		7.99E-04
BA-139		1.55E-04

TOTAL AIRBORNE TRITIUM RELEASE 8.82E+00

CAP-88 INFORMATION SHEET Source Category: Reactors Date: Anril 1991 Cooper Nuclear Station Facility: Brownsville. NE Address: Population Assessment JCL File Name: COOPER Array attached i atitude: 40° 21° 38 27 Pop. File Name: COOPER Longitude: Individual Assessment JCL File Name: Distances (meters) 600 800 1200 3200 4800 <u>6400 8000 10000 15000 20000 30000 40000 50000</u> 500 60000 80000 Food Fractions: F1 F2 F3 Meat .442 .558 .000 Urban .601 .000 Milk .399 .601 .000 Veg .700 .300 .000 Rural WEATHER DATA: STAR File Name: \_\_\_\_MKCl323 Array attached 13988 WBAN: 10 Temperature:\_ 900 meters 1323 Header: Lid Height: Code: Rainfall: \_77\_\_ cm/vr Set No: 2 STACK Source: 1 3 4 6 5 99 Height (m): Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec \_ MOMENTUM m/s ENTERED Rise Pasquill Category: Class Diam Ci/vr: Nuclide

COMMENTS: Kansas City 5- T Distant met data

File Name: COOPER

Distances

	200	300	400	500	600	800	1200	1600	3200	4800	6400
									8	5	9
NNW										6	8
NW								3		249	8
WNW									. 8	6	8
W								3	4	6	8
WSW								2	4	7	10
SW									2	123	126
SSW									3	7	10
s								2		7	10
SSE										7	
SE										3	4
ESE									4	7	54
E									6	7	9
ENE									7	7	8
NE									1	7	9
NNE										7	9

Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	10								
NNW	11								
NK	11								
WNW	11								
W	11								
WSW	13								
SW	13								
SSW	13								
S	13								
SSE	13								
SE	9								
ESE	30								
E	13								
ENE	12								
NE	10								
NNE	181								

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INSTALLATION: COOPER

LOCATION: 70 MI S OMAHA, NE

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: NEB PUB PWR & IOWA P&L DOCKET NO.: 50-298 LICENSED PWR (MWT): 2.38E+03 THERMAL PWR (MWH): 1.31E+07 NET ELECTRIC PWR (MWH): 4.20E+06 COMMERCIAL OPERATION: 07/01/74 INITIAL CRITICALITY: 02/21/74

COOLING WATER SOURCE: MISSOURI RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
KR-83M	1.38E+01
KR-85	7.64E+01
Kr ^5M .	2 70E+01
KR-87 _	8.56E+01
KR-88	8.83E+01
RB-88	8.00E-04
KR-89	3.82E+02
RB-89	5.38E-06
SR-89	4.23E-04
SR-91	6.62E-04
I-131	9.86E-03
I-132	9.25E-05
TE-132	4.08E-05
I-133	2:02E-03
XE-133	1.31E+02
XE-133M	1.15E+00
I-135	1.12E-03
XE-135	1.31E+02
XE-135M	4.72E+01
CS-137	3.84E-05
XE-137	4.60E+02
CS-138	7.61E-03
XE-138	3.74E+02
BA-139	2.09E-04
BA-140	4.59E-04
LA-140	3.14E-04

TOTAL AIRBORNE TRITIUM RELEASE N/D

CAP-88 INFORMATION SHEET Source Category: Reactors Date: #pril 1991 Dresden 2 & 3 Facility: Morris, IL Address: DRESDEN Population Assessment JCL File Name: Array attached 41 • 23 • 22 Latitude: DRESDEN 88 16 15 Pop. File Name: Longitude: Individual Assessment JCL File Name: Distances (meters) 400 500 600 300 800 1200 1600 3200 4800 6400 8000 10000 15000 20000 30000 40000 50000 60000 Food Fractions: F2 F1 **F3** Meat .008 .992 .000 Urban Milk \_000 1\_00 \_000 Rural Veg .076 .924 .000 **WEATHER DATA:** STAR File Name: ORD0452 Array attached Temperature: 10 94846 WBAN: C .950 Header: Lid Height: meters Code: 85 Rainfall: cm/yr Set No: STACK Source: 3 1 2 4 5 Height (m): Q1 Diameter (m): AREA Source Diameter (m): \_ BUOYANT cal/sec \_\_ MOMENTUM m/s X ENTERED Rise Pasquill Category:

COMMENTS: Met data Chicago/Ohare 50 miles NE of plant 5-yr

Nuclide

Class Diam Ci/vr:

### -File Names \_RESDEN

	Distances											
	200	300	400	500	600	800	1200	1600	3200	4800	6400	
N			·					3			76	
NNW								3				
NW								6			1264	
WW												
W							133					
WSW										_ 3		
· SW											947	
SSW												
S							3					
SSE								3				
SE							3		289	1100		
ESE									٦			
E								21				
ENE							3					
NE							3			1006		
NNE		_				<del></del>		3			1566	

D	2		2
u		 	99

	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	1427	_					<del></del>		
MNM									
NW									
MNM				-					
W									
WSW									
SW									
SSW					·			· · · · · · · · · · · · · · · · · · ·	
· s							<u> </u>		
SSE									
SE									
ESE									
E		,							
ENE	83					_			
NE				·					
NNE									

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INSTALLATION: DRESDEN

LOCATION: 14-MI SW TOLIET, IL

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: COMMONWEALTH EDISON
DOCKET NO.: 50-10 LICENSED PWR (MWT): 7.00E+02
THERMAL PWR (MWH): 0.00E+00 NET ELECTRIC PWR (MWH): 0.00E+00
COMMERCIAL OPERATION: 07/04/60 INITIAL CRITICALITY: 10/15/59

COOLING WATER SOURCE: KANKAKEE RIVER

UNIT NUMBER: 2 TYPE: BWR LICENSEE: COMMONWEALTH EDISON DOCKET NO.: 50-237 LICENSED PWR (MWT): 2.53E+03 THERMAL PWR (MWH): 1.45E+07 NET ELECTRIC PWR (MWH): 4.32E+06 COMMERCIAL OPERATION: 06/09/70 INITIAL CRITICALITY: 01/07/70

COOLING WATER SOURCE: KANKAKEE RIVER

UNIT NUMBER: 3 TYPE: BWR LICENSEE: COMMONWEALTH EDISON DOCKET NO.: 50-249 LICENSED PWR (MWT): 2.53E+03 THERMAL PWR (MWH): 1.36E+07 NET ELECTRIC PWR (MWH): 4.16E+06 COMMERCIAL OPERATION: 11/16/71 INITIAL CRITICALITY: 01/31/71

COOLING WATER SOURCE: KANKAKEE RIVER

AIRBORNE EFFLUENTS

NUCLIDES RELEASED

MN-54

CO-60

SR-90

CS-134

CS-137

ACTIVITY (CI)

ACTIVITY (CI)

1.39E-07

2.29E-07

3.29E-07

4.50E-05

TOTAL AIRBORNE TRITIUM RELEASE 2.60E+01

INSTALLATION: DRESDEN 2-3
LOCATION: 14 MI SW JOLIET, IL

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: BWR LICENSEE: COMMONWEALTH EDISON DOCKET NO.: 50-237 LICENSED PWR (MWT): 2.53E+03 THERMAL PWR (MWH): 1.45E+07 NET ELECTRIC PWR (MWH): 4.32E+06 COMMERCIAL OPERATION: 06/09/70 INITIAL CRITICALITY: 01/07/70

COOLING WATER SOURCE: KANKAKEE RIVER

UNIT NUMBER: 3 TYPE: BWR LICENSEE: COMMONWEALTH EDISON DOCKET NO.: 50-249 LICENSED PWR (MWT): 2.53E+03
THERMAL PWR (MWH): 1.36E+07 NET ELECTRIC PWR (MWH): 4.16E+06
COMMERCIAL OPERATION: 11/16/71 INITIAL CRITICALITY: 01/31/71

COOLING WATER SOURCE: KANKAKLE RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
CR-51	9.06E-04
MN-54	1.29E-03
CO-58	3.79E-04
FE-59	1.25E-04
CO-60	7.59E-03
ZN-65	1.50E-04
KR-85	1.30E-02
KR-85M	8.64E-01
KR-87 .	8.59E-06
KR-88	1.64E+00
SR-89	1.45E-04
SR-90	2.13E-06
ZR-95	7.80E-05
MO-99	3.66E-03
AG-110M	1.69E-05
\$B-124	2.49E-06
I-131	1.09E-01
I-133	<b>3.91E-</b> 02
XE-133	2.25E+01
I-135	6.73E-02
XE-135	1.12E+02
XE-135M	7.36E+00
CS-136	<b>5.58E-</b> 06
CS-137	4.50E-04
XE-138	2.34E+01
BA-140	5.14E-03
CE-141	4.51E-05
CE-144	3.14E-06

N/A=NOT APPLICABLE N/D=NOT DETECTED N/R=NOT REPORTED ş

## **CAP-88 INFORMATION SHEET**

Date: _Apri	1 1991			Source	Category	-React	COLE	
Facility: Address:	Duane Arno Palo, Iowa	1d						<u> </u>
Latitud	ation Assessm Array attacher le: 42 °	d 6 • 2 ·	•	Name:		ANE		
Longita	ude: 91	46 37	Pop. F	ile Name	: <u>DU</u>	ANE		_
Individ	ual Assessme	nt	JCL File	Name:				_
Distances (m 200 30 6400 800		<u>500</u> 1 <u>5000</u>	600 20000	_800 30000	1200 40000	1600 50000	3200 60000	4800 80000
Food Fraction	Meat .	F1 F2 008 .99 000 1.00 076 .92	00 .000	_	Urba Rura			
WEATHER D				R File Na	me: <u>DS</u>	M0753		_
WBAN: Header: Code: Set No:	753 DSM		nperature: Height: Yall:	10 850 78	meters cm/yr			
Height	K Source: (m): ter (m):	100	2	3 	4			3 ' — .
	Source iter (m³):							<del></del>
BUOY	ANT cal/sec ENTUM m/s							
X_ ENTE	RED Rise ill Category:	A B		<u>0</u> -	0 0 E F	<u>0</u>	meters	
Nuclide 	Class Diam	Ci/yr:						

COMMENTS Des Moines 86miles SW of plan

## - File Name: DUANE

### Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N									5	35	<b>`5</b> 5
NNW											20
NW								5	5	35	25.
WNW									20	20	25
W									20	30	40
WSW							•	4	15	.15	20
SW							-	4	10	20	25
SSW									10	240	185
S									20	20	35
SSE								3	25	85	20
SE							·		50	50	50
ESE									15	20	40
E									20	65	40
ENE									15	35	50
NE									15	30	40
NNE									15	30	45

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	50								
NNW	25								
NW	20								
WNW	35								
W	330								
WSW	90				-				
SW	25								
SSW	20								
S	15								
SSE	60								
SE	90								
ESE	65								
E	60								
ENE	50								
NE	40								
NNE	40								

INSTALLATION: DUANE ARNOLD

LOCATION: 8 MI NW CEDAR RAPIDS, IA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: IOWA ELECT LIGHT & PWR DOCKET NO.: 50-331 LICENSED PWR (MWT): 1.66E+03
THERMAL PWR (MWH): 9.97E+06 NET ELECTRIC PWR (MWH): 3.14E+06
COMMERCIAL OPERATION: 02/01/75 INITIAL CRITICALITY: 03/23/74

COOLING WATER SOURCE: CEDAR RAPIDS RIVER

AIRBORNE EFFI	LUENTS	
NUCLIDES RELE	EASED	ACTIVITY (CI)
N-13		3.15E+00
CR-51		4.73E-03
MN-54		5.18E-04
CO-58		9.52E-05
CO-60		3.29E-03
KR-85		7.45E-03
KR-85M		1.57E+01
KR-87		2.11E+01
KR-88		3.22E+01
SR-89		4.14E-05
SR-90		3.55E-07
I-131		6.65E-03
I-133		1.96E-03
XE-133		2.29E+02
XE-133M		7.30E-01
CS-134		4.00E-05
I-135		3.16E-04
XE-135		3.61E+02
XE-135M		2.29E+01
CS-137		1.41E-05
XE-138		2.06E+01
BA-LA-140		2.03E-05
CE-144		9.10E-05

TOTAL AIRBORNE TRITIUM RELEASE 2.38E+01

CAP-88 INFORMATION SHEET	
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Date: <u>April 199</u> 1	Source Cate	gory: <u>Reac</u>	tors
Facility: Edwin I. Hatch Address: Baxley, GA			
Population Assessment JCL File	Name:	EDWIN	
Latitude: 31 56 3 Pop. F.	ile Name:	EDWIN	<del></del>
Individual Assessment JCL File	Name:		
Distances (meters)  200 300 400 500 600  6400 8000 10000 15000 20000	80°0 12 30000 400	1600 1600 50000	3200 4900 60000 80000
Food Fractions: F1 F2 F3  Meat .008 .992 .000  Milk .000 1.000 .000  Veg .076 .924 .000		Jrban Rural	
WEATHER DATA:	R File Name.	AMB0771	
WBAN: 13870 Temperature: Header: 771 Lid Height: Code: Rainfall: Set No:			1
STACK Source: 1 2 Height (m): Diameter (m):	3	4 5	6
AREA Source Diameter (m):			,
BUOYANT cal/sec MOMENTUM m/s			
X ENTERED Rise 0 0 0 Pasquill Category: A B C	D E	F G	meters
Nuclide Class Diam Ci/yr			

COMMENTS: Alma met data 5-yr ~20 miles SSW of plant NOTE DAYNITE''

File Name: EDWIN

Distances

							-				
_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											25
NNW											42
NW					•						8
WNW											
W								21		21	
WSW								.9	26	9	17
SW								17	13	26	13
SSW									4	30	47
S								3	17	4	
SSE								4	9	43	26
SE										9	26
ESE											13
E						_					
ENE											
NE						-					17
NNE										4	8

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	53							·	
NNM	25		·						
NW	13					_			
WNW								•	
W	4								
WSW	13								
SW	13								
SSW	43								
S	43							•	
SSE	64								
SE	47								
ESE	13								
E	4								
ENE	13								
NE	_ 25								
NNE	4								

INSTALLATION: EDWIN I. HATCH 1&2

LOCATION: 11 MI N BAXLEY, GA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: GEORGIA PWR

DOCKET NO.: 50-321 LICENSED PWR (MWT): 2.44E+03

THERMAL PWR (MWH): 1.35E+07 NET ELECTRIC PWR (MWH): 4.11E+06 COMMERCIAL OPERATION: 12/31/75 INITIAL CRITICALITY: 09/12/74

COOLING WATER SOURCE: ALTAMAHA RIVER

UNIT NUMBER: 2 TYPE: BWR LICENSEE: GEORGIA PWR

DOCKET NO.: 50-366 LICENSED PWR (MWT): 2.44E+03
THERMAL PWR (MWH): 1.36E+07 NET ELECTRIC PWR (MWH): 4.25E+06
COMMERCIAL OPERATION: 09/05/79 INITIAL CRITICALITY: 07/04/78

COOLING WATER SOURCE: ALTAMAHA RIVER

### AIRBORNE EFFLUENTS NUCLIDES RELEASED

I-131

JCLIDES	RELEASED	ACTIVITY (CI)
AR-41		1.34E+01
CR-51		6.11E-04
MN-54		3.71E-05
CO-58		2.27E-04
CO-60		3.84E-04
ZN-65		3.57E-03
KR-85		8.81E+01
KR-85M		1.83E+02
KR-87		8.36E+01
KR-88		1.80E+02
SR-89		1.58E-03
SR-90		8.25E-06
NB-95		<b>3.65E-0</b> 5

XE-131M 6.80E+01 I-133 2.69E-02 XE-133 2.36E+03

XE-133M 2.50E+00

CS-134 1.05E-03 I-135 2.46E-02

XE-135 2.70E+02 VE-135M 1.20E+02

XE-135M 1.20E+02 CS-137 1.29E-03

XE-137 3.27E-01

XE-138 9.11E+01 BA-140 2.73E-03

BA-140 2.73E-03 LA-140 4.22E-03

#### TOTAL AIRBORNE TRITIUM RELEASE 4.89E+01

N/A=NOT APPLICABLE

N/D=NOT DETECTED

N/R=NOT REPORTED

9.39E-03

## CAP-88 INFORMATION SHEET

Date: _Apr	11 1991			Source	Category	: Ke: Ct	ors -	-	
Facility: Address:	Fermi Ur Newport							_	
	ation Assessi Array attach		JCL File	Name:	FER	MI		<del>-</del>	
Latitud Longit	<b>6</b> • 41 •	58 41 15 33	Pop. F	file Name	FER	FERMI			
Individ	ual Assessm	ent	JCL File	Name:				_	
Distances (m	neters) 00 400	_ 500	600	800	1200	1600	3200	4800	
_64co_80	<del>-1000</del>	15000	20000	30000	40000	50000	60000	80000	
Food Fraction	Meat Milk Veg	F1 F2 .008 .99 .000 1.0	000.00	 -	Urba Rura				
	Array attach	ed	STA	R File Na	une: <u>TO</u>	L0990_		_	
WBAN: Header: Code: Set No:	94830 990 TOL	Lid	nperature: Height: nfall:	9 900 <b>7</b> 9	meters				
Height	Source: (m): ter (m):	<u>51</u>		3	4				
	Source ter (m³):						<del></del>		
BUOY	ANT cal/sec ENTUM m/s							<del></del>	
Y ENTE	RED Rise ill Category:	A B	<u>c</u>	<u> </u>	e F	G '	meters		
Nuclide	Class Diam	Ci/vr:							

COMMENTS: Toleao, OH met data 15 miles SSW of plant

File Name: FERMI

	- <del></del> ±				D1:	stance	8				
	200	300	400	500	600	800	1200	1600	3200	4800	6400
N									254	133	63_
NNW									215	388	138
NW								4	25	155	314
MNM							3		47	44	76
W									50	94	181
WSW									76	864	2271
SW									225		845
SSW								68	1589	25	
s								195	250		
SSE	- · -								<del></del> .		
- SE											
ESE											
E	<del></del>										
ENE									0		
NE									142	286	11
NNE								4	162	105	49

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	130								
NNW	54								
NW	260								
WNW	65								
W	391								
WSW	1676								
SW	0								
SSW									
S									
SSE									
SE									
ESE									t.
E									
ENE									
NE					+				
NNE	60								

INSTALLATION: FERMI

LOCATION: LAGUNA BEACH, MICHIGAN

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 . AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: BWR LICENSEE: DETROIT EDISON

DOCKET NO.: 50-341 LICENSED PWR (MWT): 3.29E+03
THERMAL PWR (MWH): 1.30E+07 NET ELECTRIC PWR (MWH): 4.06E+06
COMMERCIAL OPERATION: 01/23/88 INITIAL CRITICALITY: 06/21/85

COOLING WATER SOURCE: LAKE ERIE

#### AIRBORNE EFFLUENTS NUCLIDES RELEASED ACTIVITY (CI) ALPHA 9.70E-07 NA-24 2.71E-02 CR-51 1.07E-03 MN-54 9.04E-05 CO-57 2.32E-06 CO-58 1.82E-04 CO-60 4.73E-05 ZN-65 2.72E-04 AS-76 5.07E-04 KR-85M 4.26E-01 RB-89 3.54E-04 ·SR-89 1.70E-04 SR-90 5.40E-05 2.02E-04 SR-91 Y-91M 1.62E-04 MO-99 1.53E-04 TC-99M 8.61E-04 RU-103 2.82E-05 BA-131 1.64E-04 I - 1314.05E-04 I-132 1.01E-05 I-133 6.39E-03 BA-135M 8.60E-04 I-135 8.44E-05 XE-135 6.84E-01 CS-138 1.78E-04 BA-139 1.51E-03 BA-140 9.54E-05 LA-140 2.69E-04 CE-144 6.74E-06

### TOTAL AIRBORNE TRITIUM RELEASE N/D

N/A=NOT APPLICABLE N/D=NOT DETECTED

N/R=NOT REPORTED

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Date: April 1991 CAP-88 INFORMATION SHEET  Reactors  Reactors
Facility: G: and Gulf Address: Port Gibson, MS
Population Assessment JCL File Name: GR÷ND Array attached
Latitude: 32 1 37 Longitude: 91 15 10 Pop. File Name: GRAND
Individual Assessment
Distances (meters)       200     300     400     500     600     800     1200     1600     3200     4800       6400     8000     1000     15000     20000     30000     40000     50000     60000     80000
F1 F2 F3  Meat .008 .992 .000
WEATHER DATA:  Array attached STAR File Name: JAN1169
WBAN: 03940  Header: 1169  Code: Rainfall: 134 cm/yr  Set No:
STACK Source: 1 2 3 4 5 6  Height (m): 43  Diameter (m):
AREA Source Diameter (m):
BUOYANT cal/sec
X ENTERED Rise o o o o o o meters Pasquill Category: A B C D E F G
Nuclide Class Diam Ci/yr:

COMMENTS: Jackson lo-yr met data 45 miles NE of plant

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File Name:	GRAND	GULF
------------	-------	------

					H	stance	5				
	200	300	400	500	600	800	1200	1600	3200	4800	6400
_ N_										2	
NNW								7	19		
NW								7			
MNM											
W											
WSW											
SW								4			
SSW									4	4	
S									4		4
SSE								3	7		35
SE									4	7	42
ESE								7	7	29	
E							1	3	23	42	43
ENE										19	7.
NE							1	3	4	4	12
NNE							1	10			1637

### Distances

	- 8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW									
NW									
MNM	4								
W									
WSW									
SW									
SSW									
S									
SSE	86	_							
SE	1273						_		
ESE	199					-			
E	49								
ENE	30								
NE	71								
NNE									

- · · INSTALLATION: GRAND GULF

LOCATION: 25 MI, VICKSBURG. MS

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: SYS ENERGY RESOURCES
DOCKET NO.: 50-416 LICENSED PWR (MWT): 3.83E+03
THERMAL PWR (MWH): 3.05E+07 NET ELECTRIC PWR (MWH): 9.59E+06
COMMERCIAL OPERATION: 07/01/85 INITIAL CRITICALITY: 08/18/82

COOLING WATER SOURCE: MISSISSIPPI RIVER

AIRBORNE	<b>EFFLUENTS</b>	
NUCLIDES	RELEASED	ACTIVITY (CI)
AR-41		2.02E+00
CR-51		3.11E-04
Mn-54		2.4ÒE-05
CO-58		2.57E-06
FE-59		5.40E-07
CO-60		4.72E-05
KR-85M		6.23E-01
KR-87		1.14E+00
KR-88		6.79E-01
KR-89		8.53E+01
SR-89		3.62E-06
RU-106		1.36E-05
I-131		6.07E-05
I-133		2.95E-04
XE-133		2.85E-02
XE-135		1.08E+00
XE-1351	Μ	9.21E-01
CS-137		1.22E-06
XE-138		2.56E+00

TOTAL AIRBORNE TRITIUM RELEASE 3.73E+00

N/A=NOT APPLICABLE N/D=NOT DETECTED N/R=NOT REPORTED

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## CAP-88 INFORMATION SHEET

Date:April_1991	Source Category: _Reactors
Facility: Hope Creek	
Address: Salem, NJ	
Population Assessment JCL File	Name: HOPE
Latitude: 39 • 74 • 23 •	le Name: HOPE
Individual Assessment	Name:
Distances         (meters)           200         300         400         500         600           6400         8000         10000         15000         20000	800     1200     1600     3200     4800       30000     40000     50000     60000     80000
Food Fractions: F1 F2 F3  Meat .008 .992 .000  Milk .000 1.00 .000  Veg .076 .924 .000	Urban Rural
WEATHER DATA:	R File Name: ILG1058
WBAN: 13791 Temperature Header: 1058 Lid Height: Code: ILG Rainfall: Set No:	12 C 600 meters 105 cm/yr
STACK Source: 1 2 Height (m): 59 Diameter (m):	3 4 5 6
AREA Source Diameter (m*):	
BUOYANT cal/sec MOMENTUM m/s	
X ENTERED Rise 0 0 0 C Pasquill Category: A B C	$\begin{array}{c ccccccccccccccccccccccccccccccccccc$
Nuclide Class Diam Ci/vr:	•

COMMENTS: Wilmington, DL 5-yr met data ~15 miles N of plant

File Name: HOPE

					Di	stance	S				
	200	300	400	500	600	800	1200	1600	3200	4800	6400
ที											301
NNW									_	0	3
NW											88
WNW							_			0	115
W											14
WSW				_			0		0		
SW											
SSW											14
S										0-	
SSE								0			
SE									0		
ESE											124
E										0	
ENE	0										58
NE											7_
NNE							_	<u>.</u> -			

		Distances							
	8000	10000	15000	20000	30000	40000	50000	50000	80000
N									
NNW	264		<u></u>						
NW	196								
MNM	27	-							
W	34								
WSW	14								
SW	27								
SSW									
S	20								
SSE	233								
SE	476								
ESE	556								
E	272								
ENE	82								
NE	329								
NNE	33								

INSTALLATION: HOPE CREEK.

LOCATION: 18 MI SE WILMINGTON, DE

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: PUB.SER. E&G OF NJ DOCKET NO.: 50-354 LICENSED PWR (MWT): 3.29E+03 THERMAL PWR (MWH): 2.22E+07 NET ELECTRIC PWR (MWH): 6.99E+06 COMMERCIAL OPERATION: 12/20/86 INITIAL CRITICALITY: 06/28/86

COOLING WATER SOURCE: DELAWARE RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
KR-83M	1.76E+00
KR-85M	1.76E+00
KR-87	7.\05E+00
KR-88	7.05E+00
KR-89	4.76E+01
XE-133	3.52E+00
XE-135	8.88E+00
XE-135M	1.06E+01
XE-137	5.47E+01
XE-138	3.34E+01

TOTAL AIRBORNE TRITIUM RELEASE 1.63E+02

Date:	April	1991	– CAF	'-88 INF	-ORMAI	Sou	rce Cat	egory.	Read	tors	
Facility Addre	•		s A. Fi ba. NY	tzpatri	ck						
	•	Array	attache	d		File Nar p. File N					
	Individ	u <b>al A</b> s	sessme	nt	JCL	File Nan	ne.				_
Distar 200 6,400		<u> </u>	400 10,000	500 15,00	600			200 0,000 <u>5</u>	1,600 0,000	3,200 60,000	4,800 80,000
Food	Fraction	ns:	Meat Milk		2 F3	<del></del>		Urban Rural			
WEAT	THER D		Veg attache	— - d		 Star fii	e Name:	:			
WBAt Head Code Set N	er: :	_1476 		L	emperat d Heigh Rainfall.		we	eters I/yr			
	STACH Height Diame	(m):		1 		<del></del>	3 	4	5		· ·
	AREA Diame										<del></del>
_	BUOY MOME	ANT C	al/sec // m/s								
X	ENTE: Pasqu		Rise egory:	_0	<u>0</u> B	0 0 C D	_ <u>0</u>		<u>0</u>	meters	
Nucli	<u>de</u>	Class	Diam	Ci/yr:							
СОМ	MENTS	: Roc	chester	10 yea	r met da	ata appr	oximate	ly 65 m	iles SI	of plan	nt.

-- File-Name: FITZPATR

Distances

	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											
NNW											
MN	- · · - · · · · · · · · · · · · · · · ·										
WNW											
W											
WSW								1	3		
SW									63	<u>6</u> 3	84
SSW									33	91	64
S									4	71	64
SSE									89	183	105
SE									41	109	89
ESE								3	60	22	68
E								. 3	7		
ENE											
NE											
NNE			-								

Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW				_					
NW								-	
WNW				<u>-</u>					
W									
WSW									
SW	139			_ <del></del>					
SSW	228								
S	45								
SSE	68								
SE	37								
ESE	150								
E			<u>-</u>	_					
ENE									
NE									
NNE			_	,					

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INSTALLATION JAMES A. FITZPATRICK LOCATION. 36 MI N SYRACUSE, NY

### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR

DOCKET NO · 50-333

COMMERCIAL OPERATION: 07/28/75

COOLING WATER SOURCE. LAKE ONTARIO

LICENSEE: POWER AUTHORITY, STATE OF NY

LICENSED POWER (MWT): 2.44E+03 THERMAL POWER (MWH): 1.34E+07 NET ELECTRIC POWER (MWH): 4.36E+06 INITIAL CRITICALITY: 11/17/74

AIKBORNE EI	FFLUENTS
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AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	1.34E+01
CR-51	2.28E-03
MN-54	2 26E-04
CO-58	2 42E-04
CO-60	8.45E-04
ZN-65	6.50E-05
KR-85M	4.16E+02
KR-87	1 66E+02
KR-88	5 59E+02
SR-89	5 60E-04
SR-90	4.12E-06
I-131	1.68E-02
XE-131M	`1.26E+01
I-133	4.79E-02
_ XE-133	1 41E+03
XE-133M	5.92E+01
CS-134	4.03E-05
XE-135	1.05E+03
XE-135M	5.17E+01
CS-137	1.14E-04
XE-138	1.50E+02
BA-LA-140	9 44E-04
CE-141	2.21E-07

TOTAL AIRBORNE TRITIUM RELEASE 1 11E+01

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Date: Acr	11 1991.	P-88 INFOR	MATION	Source	Categor	y: Re ct	ors	
Facility:	La Sall							
Address:	Senece.					<del></del>		
	ition Assessin Array attache		JCL File	Name:		I.,	ASALLE	
	le: 41 ude: 88		Pop. F	ile Name	e:	LASALLE		_
Individ	ual Assessme	ent	JCL File	Name:	-		<del></del>	<del>-</del> ,
Distances (m 200 30 6:00 800	eters) 0 400 0 10000	_500 	600 20000	<u>800</u> 30000	_26S _4C00C	1,600	3200	4800 -0000
Food Fraction	Meat Milk	-442 -55 399 -601	000 <u>.</u> 3	_	Urb: Rura	-		
WEATHER D	Veg . <u>ATA:</u> Array attache		STAI			ORDO4	54	_
WBAN: Header: Code: Set No:	94846 472	Lid H	perature: łeight: fall:	10 , 350 85	_ °C _ meters _ cm/yr			
Height	C Source: (m); ter (m);	<u> 113</u>	2	3	4	5 		; —
	Source ter (m²):					<del></del>		
BUOY	ANT cal/sec ENTUM m/s							
X ENTER	RED Rise ill Category:	0 B	<u>c</u>	<u>o</u> -	0 0 E F	- <u>G</u>	meters	
Nuclide	Class Diam	Ci/yr:						

COMMENTS: Let asta Chicago/Ohare 60 miles NE-of plant

File	Name:	LAS	SALLE		•	·	*	·	~	•	۰ - م
	200	300	400	500	D1s 600	tance: 800	1200	1600	3200	4800	6400
N									3		•
NNW									7	3	14
NW						_				10	45
WNW							···		14	3	19
W								3	16	10	19
WSW								3	20	7	7
SH						·	3		3	9	12
SSW								3	10	7	13
S									3	7	20
SSE				-					3	9	11
SE									3	7	14
ESE								102		10.	14
Ε								748	40	22	10
ENE									37	39	10
NE									18	10	24
AINIE											70

	Distances									
	8000	10000	15000	20000	30000	40000	50000	50000	80000	
N	6									
NNW	. 30					——————————————————————————————————————				
NW	55							<del></del>		
MVM	9									
₩	9				<del></del>					
WSW	15					•		<del></del>	<del></del>	
SW	4									
SSW	28									
S	12									
SSE	28				· · · · · · · · · · · · · · · · · · ·	·				
SE	15									
ESE										
E										
ENE					<u> </u>					
NE 2	284									
NNE	37					<del></del>				

ENE NE NNE INSTALLATION: LASALLE

LOCATION: 11 MI SE OF OTTAWA, IL

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: COMMONWEALTH EDISON DOCKET NO.: 50-373 LICENSED PWR (MWT): 3.32E+03 THERMAL PWR (MWH): 1.69E+07 NET ELECTRIC PWR (MWH): 5.44E+06 COMMERCIAL OPERATION: 01/01/84 INITIAL CRITICALITY: 06/21/82

COOLING WATER SOURCE: RESERVOIR

UNIT NUMBER: 2 TYPE: BWR LICENSEE: COMMONWEALTH EDISON DOCKET NO.: 50-374 LICENSED PWR (MWT): 3.32E+03 NET ELECTRIC PWR (MWH): 5.66E+06 COMMERCIAL OPERATION: 10/19/84 INITIAL CRITICALITY: 03/10/84

COOLING WATER SOURCE: RESERVOIR
ATROCNE EFFLUENTS

ATKBOKNE	EFFLUENTS	
NUCLIDES	RELEASED	ACTIVITY (CI)
AR-41		8.30E+00
CR-51		9.58E-04
MN-54		7.72E-04
CO-58		1.55E-04
CO-60		3.56E-03
ZN-65		4.25E-05
KR-85		4.60E-02 ·
KR-85M		5.24E+02
KR-87		2.77E-01
KR-88		6.58E+02
SR-89		5.73E-05
'MO-99		7.17E-06
I-131		7.81E-03
I-132		2.40E-02
I-133		4.02E-01
XE-133		2.42E+03
I-134		3.11E-03
Į-135		3.86E-02
XE-135		1.82E+02
CS-137		1.35E-05
LA-140		1.09E-05

TOTAL AIRBORNE TRITIUM RELEASE 3.15E-03

CAP-88 INFORMATION SHEET Source Category: Reactors Date: May 1991 Facility: Limmerick Unit 1 Address: Pottstown, PA Population Assessment JCL File Name: \_\_LIMERICK Array attached Latitude: 40 • 13 • 75 35 15 Pop. File Name: Longitude: \_\_\_\_LIMERICK Individual Assessment JCL File Name: Distances (meters): 400 200 \_\_\_300 600 500 800 1200 1600 3200 10,000 15,000 20,000 30,000 40,000 50,000 60,000 80,000 6400\_ 8000 Food Fractions: F2 F1 F3 .000 Meat .442 .558 Urban .000 .399 .601 Milk Rural 700 .300 .000 Vea WEATHER DATA: STAR File Name: \_\_\_\_RDG0184 Array attached 14712 WBAN: ဗ Temperature: \_\_\_ 12 Header: 184 900 meters Lid Height: Code: 105 RDG Rainfall: \_ cm/yr Set No: STACK Source: 1 2 3 5 6 Height (m): 61 Diameter (m): AREA Source Diameter (m<sup>3</sup>): BUOYANT cal/sec \_\_\_ MOMENTUM m/s \_\_\_\_\_ X ENTERED Rise Pasquill Category: Nuclide Class Diam Ci/vr

COMMENTS: Readgin 1-year Met Data approximately 15 miles NW of plant

File Name: LIMERICK

Distances

	200	300	400	500	600	800	1200	1600	3200	4800	6400
K							63		741	972	431
NNW								38	773	1876	1344
Nk							22		313	2034	7244
WXW							44		347	3251	12035
W							40		136	2013	1739.
WSW								53	205	612	526
Sk							79		208	235	357
SSW							13		701	354	588
S							3		393	380	188
SSE								3	218	327	3058
SE							`	6	401	153	5263
ESE								31	194	322	291
E							22		163	294	422
ENE							13		63	216	413
NE							50		44	219	363
NE							50		1182	265	519

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
	818								
NNW	1417								
N	1423								
WNW	3852								
W	1739								
WSW	526								
SW	357								
SSN	588					•			
s	390								
SSE	1822								
SE	4406								
ESE	629								
E	454								
ENE	247								
NE	300								
NNE	220								

INSTALLATION+\_LIMERICK---

LOCATION: 21 MI NW OF PHILA., PA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: PHILADELPHIA ELEC.CO
DOCKET NO.: 50-352 LICENSED PWR (MWT): 1.65E+02
THERMAL PWR (MWH): 2.19E+07 NET ELECTRIC PWR (MWH): 6.67E+06
COMMERCIAL OPERATION: 02/01/86 INITIAL CRITICALITY: 12/22/84

COOLING WATER SOURCE: SCHUYLKILL RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
KR-85	1.28E-04
SR-89	4.13E-05
I-131	5.63E-03
I-133	3.68E-03
XE-133	1.07E+02
XE-133M	9.86E-02
XE-135	4.24E+01
XE-135M	2.03E+01
UNIDENTIFIED	5.06E-06

TOTAL AIRBORNE TRITIUM RELEASE 6.40E+01

N/A=NOT APPLICABLE N/D=NOT DETECTED

N/R=NOT REPORTED

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Date: '_A	pril 1991	47 <b>-88</b> INFQ	HWYLION	Source	Category	Reac	to <sub>r</sub> s	<del></del>
Facility: Address:	Monticel Monticel							<u> </u>
Pop	ulation Assessi Array attach		JCL File	Name:	MO	NTICEL		
	urle: 45 •	19 · 59 50 · 53	Pop. F	ile Name	MO!	NTICEL		_
India	ridual Assessm	ent	JCL File	Name:		<u></u>		_
Distances 200 6400 8		500 15000	600 20000	800 30000	1200 40000	1600 50000	3200 60000	4800 80000
Food Frac	- Meat Milk	F1 F2 .008 .99 .000 1.00	000.00	_	Urba Rural			
WEATHER				R File Na	me:TO	L0990	<u>,                                     </u>	- 
WBAN: Header: Code: Set No:	14922 267	Lid	nperature. Height: nfall;	7 850 75	meters cm/yr			
Heig	CK Source: ght (m): neter (m):	100	2	3	4	5		
	A Source neter (m²):				<del>-</del>	<del></del>		
BUC	OYANT cal/sec MENTUM m/s							
X ENT	TERED Rise quill Category:	0 0 A B	<u> </u>	0 	0 0 E F	<u>G</u>	meters	
Nuclide	Class Diam	Ci/yr						

COMMENTS: Minneapolis/St. Paul 5-yr 40 miles SE of Plant .

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### Distauces

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N									5		9
NNW										9	9
NW									5	5	17
WNW										13	17
W									5	17	39
WSW								9		22	92
SW										22	78
SSW									13	_34	31
S									9	13	34
SSE								5	22	13	22
SE								5	138	1400	30:
ESE									9	17	36
E								5	5	5	255
ENE								5	9	17	216
NE										9	13
NNE									5	22	13

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	31								
NNW	309								
NW	27								
WNW	61								
W	31								
WSW	109								
SW	87								
SSW	26								
S	34								
SSE	34								
SE	87								
ESE	31								
Е	533								
ENE	312								
NE	17								
NNE	27								

INSTALLATION: MONTICELLO

LOCATION: 23 MI SE ST. CLOUD, MN

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR

DOCKET NO.: 50-263

THERMAL PWR (MWH): 1.43E+07

COMMERCIAL OPERATION: 06/30/71

LICENSEE: NORTHERN STATES PWR
LICENSED PWR (MWT): 1.67E+03

NET ELECTRIC PWR (MWH): 4.57E+06

INITIAL CRITICALITY: 12/10/70

COOLING WATER SOURCE: MISSISSIPPI RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
CR-51	4.64E-05
MN-54	1.30E-05
CO-58	8.59E-07
FE-59	4.75E-06
CO-60	2.02E-04
ZN-65	7.56E-07
KR-83M	1.27E+01
KR-85	1.69E+02
KR-85M	1.27E+01
KR-87	6.40E+01
KR-88	4.15E+01
KR-89	1.11E+03
SR-89	1.03E-03
KR-90	3.75E+01
SR-90	5.10E-06
RU-103	4.26E-07
I-131	4.31E-02
XE-131M	8.21E+00
I-133	2.12E-01
XE-133	1.62E+03
XE-133M	7.96E+00
I <b>-13</b> 5	1.93E-01
XE-135	6.20E+01
XE-135M	8.62E+01
CS-137	6.47E-04
XE-137	1.45E+03
XE-138	1.08E+03
XE-139	1.11E+02
BA-140	3.51E-03
CE-141	1.65E-05

#### TOTAL AIRBORNE TRITIUM RELEASE 8.16E+01

N/A=NOT APPLICABLE N/D=NOT DETECTED N/R=NOT REPORTED

CAP-88 INFORMATION SHEET Source Category: Reactors -Date: May-1991 --- . ---Facility: Nine Mile Point Units 1 and 2 Address: Scriba, NY Population Assessment JCL File Name: NINEMILE Array attached Latitude: <u>43</u> • 24' 36 76 4 Pop. File Name: Longitude: NINEMILE Individual Assessment JCL File Name: Distances (meters): 200 300 400 500 600 800 4800 1200 1600 3200 6400 8000 10,000 15,000 20,000 30,000 40,000 50,000 80,000 60,000 Food Fractions: F2 F1 F3 .992 Meat .008 Urban .000 .000 Rural Milk 000 1.000 .076 .924 .000 Veg WEATHER DATA: Array attached STAR File Name: \_\_\_\_ ROC0598 C WBAN: 14768 Temperature:\_ Header: Lid Height: meters 598

Set N		,	Biris	11;		_ <b>G</b> n/	уг			
_	STACK Source: Height (m): Diameter (m):	1 107		2	3		4	5		<u> </u>
	AREA Source Diameter (m²):				•		· · · · · · · · · · · · · · · · · · ·			
_	BUOYANT cal/sec MOMENTUM m/s									
<u>_x</u>	ENTERED Rise Pasquill Category:	<u> </u>	B	<u>C</u>	<u>0</u>	E		<u>0</u> 1	meters	
Nucli	de Class Diam	Ci/yr:								
<del></del>				··						

COMMENTS: Rochester 10-year Met Data approximately 65 miles SW of plant

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## File Name: NINEMILE

### Distances

	200	300	400	500	600	800	1200	1600	3200	4800	6400
N					_						
NNW					_						-
NW				_							
WNW											
W											
WSW								3			
SW								3	61	64	129
SSW									18	86	407
s									25	75	263
SSE									<b>2</b> 5	136	107
SE									111	139	87
ESE								<u> </u>	18	50	89
E									18	4	7
ENE											
NE											
NNE											

### Distances

	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW									
NW									
WNW									
W		<u></u>							
WSW									
SW	1490								
SSW	380								
S	68								
SSE	193								
SE	54	· <del></del>							
ESE	218								
E	<u>57</u>								
ENE									
NE						· .			<u> </u>
NNE									

INSTALLATION. NINE MILE POINT 1 & 2 LOCATION: 8 MI NE OSWEGO. NY

## EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1990 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE BWR LICENSEE NIAGARA MOHAWK POWER

DOCKET NO. 50-220 LICENSED POWER (MWT): 1.85E+03

THERMAL POWER (MWH) - 0.00E+00 NET ELECTRIC POWER (MWH): 0.00E+00

COMMERCIAL OPERATION: 12/01/69 INITIAL CRITICALITY: 09/05/69

COOLING WATER SOURCE: LAKE ONTARIO

UNIT NUMBER 2 TYPE BWR LICENSEE NIAGARA MOHAWK POWER DOCKET NO : 50-410 LICENSED POWER (MWT) 3.32E+03

THERMAL POWER (MWH): 8.32E+06 NET ELECTRIC POWER (MWH). 2.51E+06 COMMERCIAL OPERATION. 04/05/88 INITIAL CRITICALITY: 05/23/87

COOLING WATER SOURCE LAKE ONTARIO

AIRBORNE EFFLUENTS NUCLIDES RELEASED ACTIVITY (CI) AR-41 1 30E+01 2.29E-03 CR-51 MN-54 1 86E-04 FE-55 5.77E-04 CO-57 3 00E-04 4 28E-05 CO-58 FE-59 5.13E-05 CO-60 1 24E-03 6 73E-04 -ZN-65 SE-75 1.79E-04 4 25E-04 KR-85 4 78E+00 KR-85M KR-87 2 08E+00 KR-88 7 24E+01 SR-89 4 80E-04 2 17E-05 SR-90 MO-99 2 72E-04 I-131 1 44E-03 I-133 2.29E-02 XE-133 1.25E-05 1 89E-02 I-135 XE-135 1.06E+00 XE-135M 4 13E+00 1.68E-04 CS-137

TOTAL AIRBORNE TRITIUM RELEASE 6.34E+01

BA-140/LA-140

N/A-NOT APPLICABLE N/D-NOT DETECTED N/R-NOT REPORTED

XE-137

XE-138

4.20E+01

2 40E+01

9 02E-05

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## **CAP-88 INFORMATION SHEET**

nam	<u> 1 1991</u> -	Source Catego	ory: A ectors	· ·
Facility: Address:	Oyster Creek Toms River, Ad			· · · · · · · · · · · · · · · · · · ·
Popul	Array attached	e Name: _C	YSTER	
Latitus		File Name:	YSTER	
Individ	dual Assessment JCL Fil	e Name:		West Transport
	neters) 300 400 530 600 200 10000 15000 20000	_800_ 1200 30000 4000		
Food Fraction	Meat .008 .992 .000 Milk .000 1.00 .000		ban ıral	
WEATHER [	Veg .076 .924 .000	AR File Name: _1	ELO50	
WBAN: Header: Code: Set No:	14780 Temperature 505 Lid Height: Rainfall:			
Heigh	K Source: 1 2 t (m): eter (m):	3 =====================================	5	6
	Source eter (m²):			
BUOY	'ANT cal/sec ENTUM m/s			
Y ENTE Pasqu	RED Rise <u>0 0 0</u> <u>0</u> <u>0</u> <u>III Category: A B C</u>	0 E	o o met F G	ers
Nuclide	Class Diam Ci/vr			

COMMENTS: Lakehurst 5-yr met data approx. 15 miles Ni of plant

## File Name OYSTER

#### Distances

_	200	300	400	500	600	800	1200	1600	3200	4800_	6400
N									648	2506	710
NNW									85	777	
NW											51
WNW											
W											
WSW											10
SW									6	1	23
SSW									4	69	426
s								1	387	686	1652
SSE	,						•	1	544	480	128
SE								94	688	13	
ESE								94	655		
E									669	71	
ENE									890	970	998
NE					-			198	514	629	1098
NNE								198	743	1465	995

### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	100								
NNW									
NW									
WNW									
W									
WSW	22								
SW	761								
SSW	943	ı							
S	1300								
SSE	6								
SE									
ESE									
E									
ENE									
NE	2018								
NNE	848								

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INSTALLATION: OYSTER CREEK\_\_\_\_\_\_\_LOCATION: 9 MI S TOMS RIVER, NJ

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: JERSEY CENTRAL P&L DOCKET NO.: 50-219 LICENSED PWR (MWT): 1.93E+03 THERMAL PWR (MWH): 1.09E+07 NET ELECTRIC PWR (MWH): 3.54E+06 COMMERCIAL OPERATION: 12/01/69 INITIAL CRITICALITY: 05/03/69

COOLING WATER SOURCE: BARNEGAT BAY

AIRBORNE	<b>EFFLUENTS</b>	
NUCLIDES	RELEASED	ACTIVITY (CI)
ALPHA		1.72E-05
MN-54		5.42E-04
CO-60		1.43E-04
KR-85M		3.40E+02
KR-87		7.72E+02
KR-88		8.86E+02
SR-89		5.40E-03
SR-90		3.13E-05
TC-99M		3.70E-03
1-131		5.28E-02
I-133		2.34E-01
XE-133		9.00E+02
I-135		2.00E-01
XE-135		1.79E+03
XE-1351	1	1.61E+02
XE-138		2.04E+02
BA-140		5.60E-04
LA-140		2.87E-04

TOTAL AIRBORNE TRITIUM RELEASE 1.25E+01

N/A=NOT APPLICABLE N/D=NOT DETECTED N/R=NOT REPORTED

CAP-88 INFORMATION SHEET Source Category: Reactors May 1991 Date: Facility: Peach Bottom Units 2 and 3 Address: Peach Bottom, PA Population Assessment JCL File Name: PEACH \_\_ Array attached 39 • 45 • Latitude: Longitude: 76 • 16 9 \* Pop. File Name: PEACH Individual Assessment JCL File Name: Distances (meters): 200 300 400 500 800 1200 1600 3200 10,000 15,000 20,000 30,000 40,000 50,000 60,000 80,000 6400 8000 Food Fractions: F1 F2 F3 <u>.992</u> .008 .000 Meat Urban Milk .000 1.000 .000 Rural Veg .076 .924 .000 WEATHER DATA: ILG1058 Array attached STAR File Name: WBAN: 13791 Temperature: .12 C Header: 1058 Lid Height: 900 meters Code: ILG Rainfall: 105 \_ cm/yr Set No: STACK Source: 1 2 3 5 6 Height (m): 200 Diameter (m): AREA Source Diameter (m<sup>2</sup>): BUOYANT cal/sec MOMENTUM m/s X ENTERED RISE Pasquill Category: <u>Nuclide</u> Class Diam Ci/vr.

COMMENTS: Wilmington, DL 5-year Met Data approximately 35 miles E of plant

Pfle Name: PEACH

### Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N										89	16
NNW			<b>W</b> .					35	77	24	19
NW									141	176	246
WNW								32	32	13	58
W								6	26	38	80
WSW								3	22	35	138
SW									51	42	744
SSW								3	45	16	42
S								3	29	24	101
SSE								3	6_		36
SE											
ESE											<b>3</b> 5
E										297	43
ENE									26	77	106
NE									6	49	111
NNE									21	24	58

#### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	37								
NNW	39								
NW	207								
WNW	144								
W	314								
WSW	67								
SW	857								
SSW	83			`					
S	115								
SSE	389								
SE	50								
ESE	49								
E	80								
ENE	280								
NE	54								
NNE	47								

INSTALLATION: PEACH BOTTOM 2 & 3 LOCATION: 17 9 MI S LANCASTER, PA

#### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1990 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE- BWR LICENSEE PHILADELPHIA ELECTRIC LICENSED POWER (MWT): 3 29E+03 DOCKET NO.: 50-277 NET ELECTRIC POWER (MWH). 0.00E+00 THERMAL POWER (MWH). 0.00E+00 COMMERCIAL OPERATION. 07/05/74 INITIAL CRITICALITY 09/16/73

COOLING WATER SOURCE: CONOWINGO POND/SUSQUEHANNA RIVER

UNIT NUMBER · 3 TYPE · BWR LICENSEE PHILADELPHIA ELECTRIC DOCKET NO., 50-278 LICENSED POWER (MWT) - 3.29E+03 THERMAL POWER (MWH) · 0 00E+00 NET ELECTRIC POWER (MWH) · 0.00E COMMERCIAL OPERATION 12/23/74 INITIAL CRITICALITY 08/07/74 NET ELECTRIC POWER (MWH) · 0.00E+00

COOLING WATER SOURCE CONOWINGO POND/SUSQUEHANNA RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
NA-24	3.27E-05
KR-85M	4 50E+00
RB-88	7 41E-03
RB-89	7 85E-04
SR-89	4 43E-04
SR-90	7 55E-05
Y-91M	1 31E-03
MO-99	1 23E-04
TC-99M	1 64E-04
1-131	6 52E-03
TE-132	6 70E-06
I-133	1 42E-02
XE-133	1 57E+02
XE-135	4 54E+02
XE-135M	1 96E+01
CS-138	5 02E-02
BA-139	6 59E-03

TOTAL AIRBORNE TRITIUM RELEASE 1 94E+00

N/A-NOT APPLICABLE N/D-NOT DETECTED N/R-NOT REPORTED

Date: <u>April 199</u> 1	AP-88 INFOR	MATION -	SHEET Source	Category	Reac	tors					
Facility: Perry U Address: Perry,						······································	<del></del>				
Population Assessi		JCL File	Name:	PE	RRY		_				
	48 ' 2 "	Pop. F	ile Name	PĒ	PERRY						
Individual Assessment											
		600 20000	800 30000	1200	1600 50000	3200	4890 80000				
Food Fractions: Meat Milk Veg	F1 F2 .442 .558 .399 .600 .700 .300	F3 8 .000 1 .000 0 .000		Urba Rura	-						
WEATHER DATA:  Array attach				me: <u>IB</u>	G0745		_				
WBAN: 14843 DAY Header: 745 Code: Set No:	NITE Temp Lid H Rainf	leight:	900	meters cm/yr							
STACK Source: Height (m): Diameter (m):	1 41 -	2	3	4	5 		<u> </u>				
AREA Source Diameter (m <sup>2</sup> ):											
BUOYANT cal/sec MOMENTUM m/s											
ENTERED Rise Pasquill Category:	0 B	<u>C</u>	<u> </u>	O F	- G	meters					
Nuclide Class Diam	Ci/vr										

COMMENTS: Perry 1-yr near plant NOTE DAY: ITE!

File Name: Perry

	Distances											
	200	300	400	500	600	800	1200	1600	3200	4800	6400	
N			-									
MNM												
NW												
WNW												
W												
WSW								3				
SW								_	66	133	580	
SSW								3	60	605	332	
S								3	83	392	465	
SSE								3	440	751	416	
SE							•		416	489	218	
ESE		7							230	187	199	
E									121	339	221	
ENE	•							3	302	199	404	
NE								3				
NNE												

	Distances											
_	8000	10000	15000	20000	30000	40000	50000	50000	80000			
_ N												
NNW												
NW												
MNM				_								
W												
WSW	540											
SW	1033											
SSW	556			_								
s	489											
SSE	290											
SE	221											
ESE	501											
E	223											
ENE	1430				-,							
NE						************						
NNE				_								

INSTALLATION: PERRY

LOCATION: 7 MI NE OF PAINESVILLE, OHIO

#### EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: CLEVELAND ELEC

ILLUMINATING CO

**DOCKET NO.: 50-440** LICENSED PWR (MWT): 3.58E+03

THERMAL PWR (MWH): 2.23E+07 NET ELECTRIC PWR (MWH): 7.23E+06 COMMERCIAL OPERATION: 11/18/87

INITIAL CRITICALITY: 06/06/86

COOLING WATER SOURCE: LAKE ERIE

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
AR-41	1.03E+00
KR-85M	5.26E+01
KR-87	2.84E+00
KR-88	1.77E+01
SR-89	9.03E-04
SR-90	1.35E-04
I-131	4.51E-02
XE-131M	2.66E+00
I-132	6.64E-05
I-133	3.35E-02
XE-133	7.88E+02
XE-133M	1.46E+01
I-135	4.93E-04
XE-135	2.49E+02
XE-135M	1.18E+02
XE-137	7.97E-01
XE-138	6.68E+00

TOTAL AIRBORNE TRITIUM RELEASE 4.71E+00

N/A=NOT APPLICABLE

N/D=NOT DETECTED

N/R=NOT REPORTED

Facility:	Date: May	1991 CAP-	88 INFORM	Source (	Category:	Reactors					
Population Assessment											
	Address:	Plymouth									
Individual Assessment   JCL File Name:   PTLGRIM				L File Name:	PILGR	IM					
Distances (maters):   200		e: <u>41°</u> ude: <u>70°</u>	56' 39 ' 34' 45 ' F	op. File Neme:	PILGR	IM					
200   300   400   500   600   800   1200   1600   3200   4800   6400   8000   10,000   15,000   20,000   30,000   40,000   50,000   60,000   80,000	Individual Assessment										
Meat	200 30	0 400									
Milk	Food Fraction										
WEATHER DATA:         Array attached         STAR File Name:         NZW1144           WBAN:         14790         Temperature:         11         C           Header:         1144         Lid Height:         600         meters           Code:         NZW         Rainfall:         111         cm/yr           Set No:         1         2         3         4         5         6           Height (m):         108 <td></td> <td></td> <td></td> <td></td> <td></td> <td></td>											
Array attached   STAR File Name:   NZW1144	WEATHER D		076 .924	.000	<del></del>						
Header: 1144				STAR File Na	me:NZW1	144					
Height (m): 108 Diameter (m):	Header: Code:	1144	Lid Hei	ght: 600	meters						
Diameter (m²):  BUOYANT cal/sec	Height	(m):		2 3	4	5 6					
MOMENTUM m/s											
X ENTERED Rise 0 0 0 0 0 0 0 meters Pasquill Category: A B C D E F G	BUOY	ANT cal/sec _ ENTUM m/s _									
	X ENTE	RED Rise ill Category:	0 0 A B	0 0 C	0 0 -	neters					
Nuclide Class Diam Ci/yr:	Nuclide	Class Diam (	Di/yr:	,	,						
	<del></del>										

COMMENTS: South Weymouth 5-year Met Data approximately 25 miles NNW of plant.

### Distances

	200	300	400	500	600	800	1200	1600	3200	4800	6400
N											
NNW											
NW											
WNW						43					
W								163	18	64	1296
WSW								24	204	997	403
SW									50	144	50
SSW										20	
S								•			9
SSE									278	184	181
SE						236			534	513	902
ESE								53	564	234	
E											
ENE											
NE											
NNE											

#### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N									
NNW									
NW									
WNW	0								
W	7040								
WSW	213								
SW	145								
SSW		•							
s									
SSE	41								
SE	460								
ESE									
E									
ENE									
NE									
NNE									

INSTALLATION: PIEGRIM LOCATION: 25 MI SE BOSTON, MA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1985 AIRBORNE EFFLUENTS

UNIT NUMBER 1 TYPE. BWR
DOCKET NO.: 50-293
THERMAL POWER (MWH) · 0.00E+00
COMMERCIAL OPERATION · 12/01/72
COOLING WATER SOURCE CAPE COD BAY

LICENSEE: BOSTON EDISON
LICENSED POWER (MWT): 2.00E+03
NET ELECTRIC POWER (MWH): 0.00E+00
INITIAL CRITICALITY. 06/16/72

AIRBORNE EFFLUENTS NUCLIDES RELEASED CR-51	ACTIVITY (CI) 1.07E-06
MN-54	5.08E-05 1`58E-03
CO-60	
KR-85	2.96E-02
KR-85M	5 10E+02
KR-87	1.86E+02
KR-88	6.54E+02
SR-89	1.29E-03
SR-90	6.91E-06
1-131	4.80E-02
1-133	2 32E-01
XE-133	1.50E+03
XE-133M	9.73E+00
CS-134	6 98E-06
I-135	1.87E-01
XE-135	2.05E+02
XE-135M	5.31E+00
CS-137	6 65E-04
XE-138	2.73E+01
BA-140/LA-140	5.11E-03
CE-141	6.21E-05

TOTAL AIRBORNE TRITIUM RELEASE

6.50E+00

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N/A-NOT APPLICABLE -N/D-NOT DETECTED N/R-NOT REPORTED

Date: April 1991 Source Category: Reactors
Facility:ued Cities 1 & 2  Address:Cordovs, IL
Population Assessment JCL File Name: QUAD  Array attached
Latitude: 41 43 33 Pop. File Name: QUAD
Individual Assessment
Distances (meters)           200         300         400         500         600         800         1200         1600         3200         4800           6400         8000         10000         30000         40000         50000         60000         80000
Food Fractions: F1 F2 F3  Meat .442 .558 .000  Milk .399 .601 .000  Rural
Veg .700 .300 .000  WEATHER DATA:  Array attached STAR File Name: MLI0269
WBAN: 14923 Temperature: 10 C Header: 269 Lid Height: 950 meters Code: ELI Rainfall: 29 cm/yr Set No:
STACK Source: 1 2 3 4 5 6  Height (m):
AREA Source Diameter (m):
BUOYANT cal/sec
X ENTERED Rise 0 0 0 0 0 0 meters Pasquill Category: A B C D E F G
Nuclide Class Diam Ci/vr:

COMMENTS: Moline/Quad City 5-yr 20 miles SSW of plent

, \_File Name: QUAD

Distances												
200	300	400	500	600	800	1200	1600	3200	4800	6400		
						14		45	10	5		
									5	10		
									5	10		
									30	35		
									10	10		
									10	50		
									20	75		
								5	25	210		
								10	50	100		
							25	25	30	551		
							10	10	5	80		
		,								10		
									2	10		
							2	5	10	5		
							2	15	10	5		
							2	25	10	40		
	200	200 300			200 300 400 500 600	200 300 400 500 600 800	14	200 300 400 500 600 800 1200 1600  14  25 10	200 300 400 500 600 800 1200 1600 3200  14 45  5 10 25 25 10 10	200         300         400         500         600         800         1200         1600         3200         4800           10         14         45         10         5         5         30         10         10         10         10         10         10         10         10         10         50         25         25         30         10         50         25         25         30         10         5         3         2         2         3         2         2         5         10         2         5         10         2         5         10         10         5         3         2         2         5         10         2         15         10         10         10         5         3         3         2         2         5         10		

			Distances									
	8000	10000	15000	20000	30000	40000	50000	60000	80000			
n <sup>-</sup>	32C					<u> </u>		•				
NNW	50											
NW	50	`										
MNW	100											
W	230					1						
WSW	60											
SW	90											
SSW	681											
S	210											
SSE	40											
SE	10											
ESE	5											
E	10											
ENE	210											
NE	320											
NNE	2205											

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INSTALLATION: QUAD-CITIES LOCATION: 20 MI NE MOLINE, IL

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: COMMONWEALTH ED &

IOWA-ILLINOIS G&E

DOCKET NO.: 50-265 LICENSED PWR (MWT): 2.51E+03
THERMAL PWR (MWH): 1.83E+07 NET ELECTRIC PWR (MWH): 5.66E+06
COMMERCIAL OPERATION: 02/18/73 INITIAL CRITICALITY: 10/18/71

COOLING WATER SOURCE: MISSISSIPPI RIVER

UNIT NUMBER: 2 TYPE: BWR LICENSEE: COMMONWEALTH

ED&IOWA-ILLINOIS GAS&ELEC

DOCKET NO.: 50-265

THERMAL PWR (MWH): 1.36E+07

COMMERCIAL OPERATION: 03/10/73

LICENSED PWR (MWT): 2.51E+03

NET ELECTRIC PWR (MWH): 4.18E+06

INITIAL CRITICALITY: 04/26/72

COOLING WATER SOURCE: MISSISSIPPI RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
CR-51	2.53E-03
MN-54	4.18E-04
CO-58	1.51E-04
CO-60	6.78E-03
ZN-65	3.06E-05
KR-88	1.69E+00
SR-89	3.85E-04
SR-90	3.70E-06
MO-99	1.82E-03
AG-110M	9.59E-06
I-131	6.07E-03
I-133	3.56E02
XE-133	1.72E+00
I-135	9.14E-02
XE-135	3.56E-01
CS-137	7.75E-04
BA-140	2.91E-04
LA-140	9.70E-04
HF-181	8.76E-06

TOTAL AIRBORNE TRITIUM RELEASE 4.50E+01

N/A=NOT APPLICABLE N/D=NOT DETECTED N/R=NOT REPORTED

CAP-88 INFORMATION SHEET Date: April 1991 Source Category: Reactors River Bend Facility: St, Francisvile, LA Address: RIVER Population Assessment JCL File Name: Array attached 30 45 25 4 Latitude: 91 • 19 • 54 • Pop. File Name: RIVER Longitude: Individual Assessment JCL File Name: Distances (meters) 200 800 1200 1600 3200 4800 500 600 300 400 10000 15000 20000 30000 40000 50000 60000 80000 <u>6400 8000</u> Food Fractions: F2 F1 **F3**  
 Meat .008 .992 .000

 Milk .000 1.000 .000

 Veg .076 .924 .000
 Urban Rural WEATHER DATA: STAR File Name: BTR0166 Array attached Temperature: 20 Lid Height: 900 WBAN: 13970 C 166 Header: Lid Height: meters 152 Code: Ramaii: cm/yr Set No: STACK Source: 1 2 3 5 6 Height (m): 59 Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec MOMENTUM m/s X ENTERED Rise Pasquill Category: Nuclide Class Diam Ci/vr:

COMMENTS: Baton Rouge 10-yr met data 25 miles SSE of plant

File Name: RIVER

Distances

							_				
_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N									91	11	28
NNW									40	170	9
NW								4	21	150	45
WNW								16	68	33	735
W								12	6	51	588
WSW											
SW										13	
SSW											
S			•					·			
SSE								4	3_		
SE										25	
ESE										16	15
E									3	11	21
ENE									11	6	40
NE									42	12	12
NNE								9	37	16	

Distances

N 16 NNW 163 NW 16 WNW 104 W 231 WSW 14 SW SSW S 26 SSE SE ESE 10	80000	60000	50000	40000	30000	20000	15000	10000	8000	_
NW 16  WNW 104  W 231  WSW 14  SW  SSW  S 26  SSE  SE  ESE 10							—		16	N
WNW 104  W 231  WSW 14  SW  SSW  S 26  SSE  SE  ESE 10									163	NNW
W 231 WSW 14 SW SSW S 26 SSE SE ESE 10									16	NW
WSW 14 SW SSW S 26 SSE SE ESE 10									104	WNW
SW									231	W
SSW						-			14	WSW
\$ 26 \$SE \$E ESE 10										SW
SSE SE ESE 10				•						SSW
SE ESE 10									26	S
ESE 10										SSE
										SE
									10	ESE
£ 61									61	E
ENE 10									10	ENE
NE 49									49	NE
NNE 28									28	NNE

-- INSTALLATION: RIVER BEND

LOCATION: 24 MI NNW OF BATON ROUGE, LA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 . AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: GULF STATES UTILITIES DOCKET NO.: 50-458 LICENSED PWR (MWT): 2.89E+03
THERMAL PWR (MWH): 2.25E+07 NET ELECTRIC PWR (MWH): 7.25E+06
COMMERCIAL OPERATION: 06/16/86 INITIAL CRITICALITY: 10/31/85

COOLING WATER SOURCE: MISSISSIPPI RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
MN-54	3.09E-04
CO-58 ·	6.72E-06
CO-60	3.89E-05
SR-89	1.47E-05
I-131	4.78E-04
I-133	4.76E-03
XE-135	2.05E+00

#### TOTAL AIRBORNE TRITIUM RELEASE 2.88E+00

N/A=NOT APPLICABLE N/D=NOT DETECTED N/R=NOT REPORTED

## CAP-88 INFORMATION SHEET

Date: <u>April 1991</u>	Source Category: Resectores
Facility: Sus uehanna 1 & 2  Address: Berwick, PA	
Array attached Latitude: 41 5 59	Name: SUSQUEHA
Longitude: 76 9 9 Pop. F	ile Name:
Individual Assessment	Name:
Distances (meters) 200 300 400 500 600	800 1200 1600 3200 4800
6400 8000 15000 20000	30000 40000 50000 60000 P000C
Food Fractions: F1 F2 F3  Meat .008 .922 .000  Milk .000 1.00 .000  Veg .076 .924 .000	Rural
WEATHER DATA:	R File Name: AVP0499
WBAN: 14777 Temperature Header: 499 Lid Height: Code: AVP Rainfall: Set No:	
STACK Source: 1 2 Height (m): 60 Diameter (m):	3 4 5 6
AREA Source Diameter (m):	
BUOYANT cal/sec MOMENTUM m/s	
Pasquill Category: A B C	D E F G meters
Nuclide Class Diam Ci/vr:	

COMMENTS: Wilkes-Barre 5-yr met data 20 miles NE of rlant

### File Name: SUSQUENA-

#### Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N							_	40	15	49	1159
NNW							20		19	15	
NM								16_	73	94	
MMM							12		42	45	103
W								4	31	60	96
WSW								4	61	340	116
SW								4	15	344	93
SSW		•						4	249	45	86
\$								36	150	30	142
SSE							40		157	38	93
SE		·				45			219	151	140
ESE							24		27	64	182
E									57	23	43
ENE									15	38	70
NE										166	199
NNE									17	30	

Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
N	2215								
NNW	23								
NW	97								
WNW	267								
W	187								
WSW	4181	·							
SW	1560								
SSW	123								
S	10								
SSE	47								
SE	13								
ESE	177								
E	79								
ENE	97								
NE	379								
NNE									

INSTALLATION: SUSQUEHANNA

LOCATION: 7 MI NE OF BERWICK, PA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR LICENSEE: PENNSYLVANIA P&L
DOCKET NO.: 50-387 LICENSED PWR (MWT): 3.29E+03
THERMAL PWR (MWH): 2.65E+07 NET ELECTRIC PWR (MWH): 8.41E+06
COMMERCIAL OPERATION: 06/08/83 INITIAL CRITICALITY: 09/10/82

COOLING WATER SOURCE: SUSQUEHANNNA RIVER

UNIT NUMBER: 2 TYPE: BWR LICENSEE: PENNSYLVANIA P&L
DOCKET NO.: 50-388 LICENSED PWR (MWT): 3.29E+03
THERMAL PWR (MWH): 1.87E+07 NET ELECTRIC PWR (MWH): 5.90E+06
COMMERCIAL OPERATION: 02/12/85 INITIAL CRITICALITY: 05/08/84

COOLING WATER SOURCE: SUSQUEHANNNA RIVER

AIRBORNE EF	FLUENTS	
NUCLIDES RE	LEASED	ACTIVITY (CI)
CR-51		7.37E-04
MN-54		2.74E-04
CO-58		3.95E-05
CO-60		5.50E-05
SR-89		7.81E-07
I-131		7.07E-04
XE-133		7.25E+01

TOTAL AIRBORNE TRITIUM RELEASE 2.05E+01

N/A=NOT APPLICABLE N/D=NOT DETECTED N/R=NOT REPORTED

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## **CAP-88 INFORMATION SHEET**

Date:	_April_	1991			Source	Cate	gory:	<u> Res</u>	ctors		•
Facility Addre		Vermont Vernon,									
	Population	Assessmay attache		JCL File	Name:	•	VERM	ont			
	Latitude: Longitude	42*	48 4	9° 6° Pop. F	ile Nam	e: ,	VERM	ont			
	Individual	Assessme	ent	JCL File	Name:	• •					
200	300 8000	400	500 15000	600 20000	800 30000		00	1600 50000	32 600		4800 0000
	Fractions:	Milk Veg	.000 1.	92 .000			Urban Rural				
WEAT	THER DATA	<u>4:</u> ay attache	ed	STA	R File N	lame:	CEF	0182	<del></del>		
WBAI Head Code Set N	er:	703 182 CEF	Lic	mperature i Height: infall:	1000	_ 11161					
	STACK So Height (m Diameter	):	94	2	3		4		5 	6	- -
	AREA So Diameter										-
	BUOYAN										<b>-</b>
<u>x</u>	ENTERED Pasquill (		O C	0 B C	<u>O</u>	<u>0</u>	0 F	<u>G</u>	mete	ers	
Nucli	de Cla	ess Diam	Ci/yr-								<u>-</u> 
COM	IMENTS: (	Chicope	Falls	/Westove	r 5-y	raet	data	45	mıle	es of t	lant

File Name: VERMONT

					Di	stance	:s			•	-
_	200	300	400	500	600	800	1200	1600	3200	4800	6400
N									31	104	224
NNW									31	212	104
NW								11	31	15	175
WNW								59	31	31	31
W							3			108	6
WSW			•				6		6	21	46
SW							42		31	6	46
SSW				·			7		36	6	21
S							3		92	139	21
SSE			·						73	21	71
SE							•		73	62	130
ESE									15	10	31
E								165	335	211	139
ENE				. = =				236	731	288	10
NE								11	36	98	6
NNE									15	- 36	52

#### Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
	42								
NNW	2600								
NW	324								
WNW	139								
W	114								
WSW	36								
SW	91								
SSW	6								
S	21								
SSE	87								
SE	242								<u> </u>
ESE	15								
E	472				-				
ENE	25								
NE	21								
NNE									

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INSTALLATION: VERMONT YANKEE LOCATION: 5 MI S BRATTLEBORO, VT

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 1 TYPE: BWR

DOCKET NO.: 50-271

THERMAL PWR (MWH): 1.30E+07

COMMERCIAL OPERATION: 11/30/72

LICENSEE: VERMONT YANKEE NUC. PWR
LICENSED PWR (MWT): 1.59E+03

NET ELECTRIC PWR (MWH): 4.11E+06

INITIAL CRITICALITY: 03/24/72

COOLING WATER SOURCE: CONNECTICUT RIVER

AIRBORNE	<b>EFFLUENTS</b>	
NUCLIDES	RELEASED	ACTIVITY (CI)
MN-54		9.82E-05
CO-58		9.78E-07
CO-60		2.54E-03
2N-65		7.99E-05
SR-89		1.12E-04
SR-90		4.61E-06
I-131		3.47E-03
I-133		4.97E-03
CS-134		2.78E-05
CS-137		2.21E-04

#### TOTAL AIRBORNE TRITIUM RELEASE 4.39E+01

N/A=NOT APPLICABLE N/D=NOT DETECTED

N/R=NOT REPORTED

**CAP-88 INFORMATION SHEET** Date: \_april 1991 Source Category: Re-ctors Washington Nuclear Project #2 Facility: Richland, WA Address: Population Assessment JCL File Name: WPN2 Array attached 29 Pop. File Name: Latitude: 46° 16 ' 119. 17. WPN2 Longitude: Individual Assessment JCL File Name: Distances (meters) 500 600 800 1200 1600 3200 4800 400 200 300 10000 15000 20000 30000 40000 50000 60000 6400 8000 80000 Food Fractions: F1 F2 F3 Meat .008 .992 .000 Urban Milk .000 1.00 .000 Rural Veg .076 .924 .000 **WEATHER DATA:** \_ Array attached STAR File Name: YKM0484 24243 WBAN: Temperature:\_ 1100 meters 484 Header: Lid Height: Code: YKM Rainfall: 42 cm/yr Set No: STACK Source: 3 2 4 5 6 70 Height (m): Diameter (m): AREA Source Diameter (m): BUOYANT cal/sec MOMENTUM m/s Y ENTERED Rise Pasquil Category: A B C D E F G

COMMENTS: Yakıra 5-vr met q-ta 25 miles E of plant

Class Diam Ci/yr

Nuclide

File Name. WNP2

. . . . . .

Distances

_	200	300	400	500	600	800	1200	1600	3200	4800	6400
K								0,	0		
NNW							0		•		
NW	-		0				0				
WNK							.,			0	0
W							0				0
WSW									0.		
SW											
SSW						0		0			
S						•		0		0	0
SSE									0	0	0
SE							0		0	0	
ESE								0	0		
E											
ENE							0				20
NE				0		*****		0	0		
NNE									0	0	

Distances

_	8000	10000	15000	20000	30000	40000	50000	60000	80000
ĸ									
NNW									
NW	0								
WNW									
W									
WSW									
SW	4								
SSW	140								
S	12								
SSE	36	· · · · · · · · · · · · · · · · · · ·							
SE	24								
ESE	52								
E	96								
ENE	96								
NE	4								
NNE									

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INSTALLATION: WNP-2

LOCATION: 12 MI NW OF RICHLAND, WA

# EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT FOR YEAR 1988 AIRBORNE EFFLUENTS

UNIT NUMBER: 2 TYPE: BWR LICENSEE: WASHINGTON PUBLIC PWR

SUPPLY SYSTEM

DOCKET NO.: 50-397

THERMAL PWR (MWH): 2.06E+07

COMMERCIAL OPERATION: 12/13/84

LICENSED PWR (MWT): 3.32E+03

NET ELECTRIC PWR (MWH): 6.54E+06

INITIAL CRITICALITY: 01/19/84

COOLING WATER SOURCE: COLUMBIA RIVER

AIRBORNE EFFLUENTS	
NUCLIDES RELEASED	ACTIVITY (CI)
NA-24	2.87E-03
AR-41	6.18E-01
CR-51	1.54E-02
MN-54	7.00E-04
MN-56	9.30E-04
CO-58	2.73E-03
CO-60	6.81E-03
CU-64	1.44E-02
2N-65	4.27E-02
ZN-69M	7.06E-04
AS-76	4.80E-05
BR-82	1.99E-05
KR-85	3.40E-02
KR-85M	4.39∑+01
KR-87	1.66E+01
KR-88	5.20E+01
SR-89	8.81E-05
SR-90	1.88E-06
ZR-NB-95	1.31E-04
MO-99	1.20E-03
TC-99M	8.87E-03
AG-110M	4.20E-06
I-131	9.00E-Q2
XE-131M	6.23E+00
I-132	6.74E-03
I-133	4.02E-02
XE-133	5.64E+02
XE-133M	4.00E+01
CS-134	4.50E-04
I-135	1.36E-03
XE-135	1.15E+02
XE-135M	4.51E+01
CS-137	2.80E-04
CS-138	2.49E-01
XE-138	1.65E+01
BA-139	1.00E-02

TOTAL AIRBORNE TRITIUM RELEASE 8.59E+00

INSTALLATION: WNP-2

N/A=NOT APPLICABLE N/D=NOT DETECTED N/R=NOT REPORTED Page Intentionally Blank

## APPENDIX D

## LISTS OF NUCLEAR POWER PLANTS

TABLE D-1: NUCLEAR POWER PLANTS IN THE UNITED STATES

	INIT CONN									
					4057		ž	CRIT	CONN	
44-	tet	TI ITY/INIT	CITY	CTATE	NET Mue	TYPE	COMPL (1		START	
No.	U	TLITY/UNIT	CITY	STATE	INC	ITPE	conrt (1	, MIE	DATE	
	AL	ABANA POWER CO.								
1	•	farley 1, Joseph M	Dothan	Ala.	824	PUR	100	08/1977	12/1977	
2	*	Farley 2, Joseph H	Dothan	Ala	828	PUR	100	05/1981	07/1981	
	AF	ZIZONA PUBLIC SERVICE CO								
3	•	Palo Verde 1	Wintersburg	Arız.	1221	PUR	100	<b>05/198</b> 5	01/1986	
4	*	Palo Verde 2	Unntersburg	Arız.	1221	PUR	100	04/1986	09/1986	
5	*	Palo Verde 3	Vintersburg	Artz.	1221	PUR	100	10/1987	01/1988	
	84	LTIMORE GAS & ELECTRIC CO								
6	*	Calvert Cliffs 1	Lusby	nd.	<b>82</b> 5	PUR	100	10/1974	05/1 <del>9</del> 75	
7	*	Calvert Cliffs 2	Lusby	Md.	825	PUR	100	11/1976	04/1977	
	Ð	STON EDISON CO.								
8	*	Pilgris	Plymouth	Mass.	670	BUR	100	06/1972	12/1972	
	CA	AROLINA POWER & LIGHT CO								
9	•	Brunswick 1	Southport	R C.	790	BUR	100	10/1976	03/1977	
10	•	Brunswick 2	\$outhport	N.C.	790	BUR	100	03/1975	11/1975	
11	*	NOO HISON &	Hartsville	N.C.	<b>66</b> 5	PUR	100	09/1970	03/1971	
12	*	Shearon Harris	Hew Hill	N.C	860	PUR	100	01/1987	05/1987	
	TH	E CLEVELAND ELECTRIC ILLUMINA	TING CO							
13	•	Perry 1	North Perry	Ohto	1141	BUR	100	06/1986	11/1987	
14		Perry 2	North Perry	Ohio	<b>120</b> 5	BUR	57	Indefini	te	
		MHONWEALTH EDISON CO								
15	*	Brandwood 1	Brandwood	ILL.	1120	PUR	100	05/1987	07/1988	
16	•	Di di tempor L	Brandwood	111	1120	PUR	100	03/1988	10/1968	
17	•	Byron 1	Byron	111	1105	PUR	100	02/1985	09/1985	
18	*	D). G. E	Byron	Itt.	1105	PWR	100	01/1987	08/1987	
19	*	Dresden 2	Norris	Itt.	772	BUR	100	01/1970	06/1970	
20	•	Dresden 3	Morr1\$	111	772	BUR	100	01/1971	11/1971	
21		LaSalle County 1	Seneca	III.	1036	BUR	100	06/1982	01/1984	
22		LaSalle County 2	Seneca	ILL.	1036	BUR	100	03/198%	10/1984	
23		Quad Cities 1	Cordova	ILL.	769	BUR	100	10/1971	03/1973	
24	*		Cordova	111.	769	BUR	100	04/1972	03/1973	
25	*	Zion 1	Zion	Ill	1040	PVR	100	06/1973	12/1973	
26	*	Z10n 2	Zion	III.	1040	PUR	100	12/1973	09/1974	
		MINECTICUT YANKEE ATOMIC POWER			* **					
27	*	Haddam Neck	Haddam Neck	Conn	<b>56</b> 5	PUR	100	07/1967	01/1968	
		DISOLIDATED EDISON CO					40-	00.00	AD 15071	
28	*	Indian Point 2	Indian Point	N Y.	970	PUR	100	05/1973	08/1974	

		TABLE D-1. NUCLEAR POWER PLANTS IN THE UNITED STATES (Continued)						
							INIT	COMM
				NET		×	CRIT	" START
No	UTILITY/UNIT	CITY	STATE	HVe	TYPE	COMPL (1)	DATE	DATE
	CONSUMERS POWER CO.							
29	* Big Rock Point	Charlevoix	Mach	67	BUR	100	09/1962	03/1963
30	* Patisades	South Haven	Mich	768	PUR	100	05/1971	12/1971
	DETROIT EDISON CO							
31	* ferm: 2	Newport	Kich	1075	BWR	100	06/1985	01/1988
`	DUKE POWER CO.							
32	* Catawba 1	Clover	s c	1129	PUR		01/1985	06/1985
33	* Catawba 2	Clover	S C	1129	PUR		05/1986	<b>08/198</b> 6
34	* McQuire 1	Cornelius	N C	1129	PUR	100	08/1981	12/1981
35	* McQuare 2	Cornel 1us	N -	1129	PWR	100	05/1983	03 40.4
<b>3</b> 6	* Oconee 1	Seneca	s c	846	PUR	100	04/1973	07/1973
37	* Oconee 2	Seneca	S C.	<b>84</b> 6	PWR	100	11/1973	09/1974
38	* Oconee 3	Seneca ,	s.c.	846	PWR	100	09/1974	12/1974
	DUQUESNE LIGHT CO.	-						
39	* Beaver Valley 1	Shippingport	Pa.	810	PUR	100	05/1976	10/1976
40	* Beaver Vailey 2	Shippingport	Pa	830	PUR	100	08/1987	
	ENTERGY OPERATIONS, INC.							
41	* Arkansas Nuclear One 1	Russellville	Ark.	<b>83</b> 6	PWR	100	08/1974	12/1974
42	* Arkansas Nuclear One 2	Russetlville	Ark	<b>8</b> 58	PWR	100	12/1978	03/1980
43	* Grand Gulf	Port Gibson	HISS	1142	BUR	100	08/1 <del>9</del> 82	07/1985
44	* Waterford 3	Taft	La	1075	PWR	100	03/1985	09/1985
	FLORIDA POWER & LIGHT CO							
45	* St. Lucie 1	Hutchinson Island	Fla	<b>83</b> 9	PUR	100	04/1976	12/1976
46	* St Lucie 2	Hutchinson Island	Fla	839	PUR	100	06/1983	08/1983
47	* Turkey Point 3	Florida City	FLA	666	PWR	100	10/1972	12/1972
48	* Turkey Point 4	Florida City	Fla	666	PUR	100	06/1973	09/1973
	FLORIDA POWER CORP							
49	* Crystal River 3	Red Level	Fla	821	PWR	100	01/1977	03/1977
	GEORGIA POWER CO.							
50	* Hatch 1, Edwin I	Baxley	Çe.	753	BUR		09/1975	12/1975
51	* Hatch 2, Edwin I	Baxley	60.	<b>76</b> 6	BUR	100	07/1978	09/1979
52	* Vogtle 1, Alvın W.	Waynesboro	<b>6</b>	1079	PUR	100	03/1987	06/1987
53	* Vogtle 2, Alvın W.	Waynesboro	44	1100	PWR	100	03/1989	05/1989
	GPU NUCLEAR CORP.							
54	* Oyster Creek	Forked River	H J	620	BWR		05/1969	12/1969
55	* Three Mile Island 1	Londonderry Twp	Pa	<b>808</b>	PUR	100	06/1974	09/1974
	GULF STATES UTILITIES CO							
56	* River Bend	St Francesville	La.	936	BUR	100	10/1985	06/1986

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TABLE 0-1: NUCLEAR POWER PLANTS IN THE UNITED STATES (Continued)

	'^	BUE U-1: MULLEAK P	OWER PLANIS	TH THE ONT	. 150 317	iles (ront		
				NET		z	INIT	CONN
No.	UTILITY/UNIT	CITY	STATE	file	TYPE	COMPL (1	CRIT DATE	START DATE
	0.36211/0.021	C111	JIAIL	1120		COI! C (1	, DAIL	DATE
	HOUSTON LIGHTING & POWER CO.							
57	* South Texas Project 1	Palacios	Tex	1250	PUR	100	03/1988	08/1988
58	* South Texas Project 2	Palacios	Tex	1250	PUR	<b>10</b> 0	03/1989	06/1989
	ILLINOIS POWER CO.							
59	* Clinton	Clanton	111	930	BUR	100	02/1987	04/1987
	INDIANA/MICHIGAN POWER CO							
60	* Cook 1, Donald C.	Bridgman	Hich.	1020	PUR	100	01/1975	08/1975
61	* Cook 2, Donald C	Bridgman	Hich	1060	PUR	100	03/1978	07/1978
	10NA ELECTRIC LIGHT & POWER CO.		_					
62	* Duane Arnold	Palo	Iova	538	BUR	100	03/1974	02/1975
	MAINE YANKEE ATOMIC POWER CO.							
63	* Name Yankee	Wiscasset	He	830	PUR	100	10/1972	12/1972
	NEBRASKA PUBLIC POWER DISTRICT							
64	* Cooper	Brownville	Neb.	764	BWR	100	02/1974	07/1974
	NEW HAMPSHIRE YANKEE							
65	\$eabrook	Seabrook	N.H.	1150	PUR	100	06/1989	1990
	NEW YORK POWER AUTHORITY							
<b>6</b> 6	* Fitzpatrick, James A	Scriba	N.Y.	757	BUR	100	11/1974	07/1975
67	* Indian Point 3	Indian Point	NY	965	PUR	100	04/1976	<b>08/197</b> 6
	NIAGARA NOHAWK POWER CO							
68	* Nine Hile Point 1	Scraba	N.Y.	610	BWR	100	09/1 <del>969</del>	12/1969
69	* Nine Kile Point 2	Scriba	N.Y.	1072	BWR	100	05/1987	04/1988
	NORTHEAST UTILITIES							
70	* Hillstone 1	Waterford	Conn.	654	BUR	100	10/1970	06/1971
71	* Hillstone 2	<b>Vaterford</b>	Conn	<b>863</b>	PUR	100	10/1975	12/1975
72	* Hillstone 3	Water lord	Conn	1142	PUR	100	01/1986	04/1986
	MORTHERN STATES POWER CO.							
73	* Monticello	Montroello	Bann	536	BWR	100	12/1970	06/1971
74	* Prairie Island 1	Red Ving	Hann.	503	PUR	100	12/1973	12/1973
<b>7</b> 5	* Prairie Island 2	Red Wing	Hino	500	PWR	100	12/1974	12/1974
	OMAHA PUBLIC POWER DISTRICT							
76	* Fort Calhoun	Fort Calhoun	Neb	478	PUR	100	08/1973	06/1974
	PACIFIC GAS & ELECTRIC CO.							
77	* Diablo Canyon 1	Avila Beach	Cal	1073	PWR	<b>10</b> 0	04/1984	05/1985

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TABLE D-1. NUCLEAR POWER PLANTS IN THE UNITED STATES (Continued)

		TABLE D-T. NUCLEAR P	OWER PLANIS	IN THE UN	ITED STA	ITES (Cont		
							INIT	COMM
No	HTT: TTV //MITT	*****		NET		2	CRIT	START
MO	UTILITY/UNIT	CITY	STATE	Mue	TYPE	COMPL (1	) DATE	DATE
78	* Diablo Canyon 2	Avila Beach	Cal	1087	PUR	100	08/1985	03/1986
	PENNSYLVANNIA POWER & LIC	SHT CO.						
79	* Susquehanna 1	Berwick	Pa	1032	BUR	100	09/1983	06/1983
80	* Susquehanna 2	Berwick	Pa.	1038	BUR	100	05/1984	02/1985
	PHILADELPHIA ELECTRIC CO				•			
81	* Limerick 1	Pottstown	Pa.	1055	SUR	100	12/1984	02/1986
82	* Limerick 2	Pottstown	Pa.	1055	BUR	100	08/1989	02/1990
83	* Peach Bottom 2	Peach Bottom	Pa	1051	BUR	100	09/1973	07/1974
84	* Peach Bottom 3	Peach Bottom	Pa.	1035	BUR	100	08/1974	12/1974
	PORTLAND GENERAL ELECTRIC							
85	* Trojan	Prescott	Ore	1095	PUR	100	12/1975	05/1976
	Bilbi 10 ochurer expens							
86	PUBLIC SERVICE ELECTRIC &  * Hope Creek							
87	* Salem 1	Salem	N.J.	1031	BUR	100	06/1986	12/1986
<b>8</b> 8	* Salem 2	Satem	N.J	1106	PUR	100	12/1976	06/1977
20	- Satem 2	Salem	N.J.	1106	PUR	100	08/1980	10/1981
	ROCHESTER GAS & ELECTRIC	CORP						
89	* Ginna, R. E.	Ontario	R f.	470	PWR	100	11/1969	07/1970
	SOUTH CAROLINA ELECTRIC 8	GAS CO.						
90	* Summer, Virgil C	Parr	s.c.	885	PUR	100	10/1982	01/1984
	SOUTHERN CALIFIRNIA EDISO	at 2.						
	SAN DIEGO GAS & ELECTRIC	-						
91	* San Onofre 1	San Clemente	Cal	436	PUR	100	04 /1047	VA 14010
92	* San Onofre 2	San Clemente	Cai.	1070	PUR	100	06/1967 07/1982	`01/1968
93	* San Onofre 3	San Clemente	Cal.	1080	PUR	100	08/1983	08/1983 04/1984
	TEMMECORE WALLEY AND AND	u.						
94	TENNESSEE VALLEY AUTHORIT Bellefonte 1	•	••-	4849				
95	Beliefonte 2	Scottsboro	Ala.	1213	PUR		Indefinit	
96	* Browns Ferry 1	Scottsboro	Ala	1213	PVR	58	Indefinit	
97	* Browns Ferry 2	Decatur	Ala.	1065	BUR	100	08/1973	08/1974
98	* Browns Ferry 3	Decatur	Ala	1065	BUR	100	07/1974	03/1975
99	* Sequoyah 1	Decatur Soddw-Datey	Ala	1065	BUR	100	08/1976	03/1977
100	* Sequoyah 2	Soddy-Daisy	Tenn.	1148	PUR	100	07/1980	07/1981
101	Watts Bar 1	Soddy~Daisy Spring City	Tenn	1148	PWR	100	11/1981	06/1982
102	Watts Bar 2		Tenn	1177	PWR	100 85	1-de4	06/1992
	Antia hou C	Spring City	Tenn	1177	PUR	60	Indefinit	i e
	TEXAS UTILITIES ELECTRIC	Co						
103	Comanche Peak 1	* Glen Rose	Tx.	1150	PUR	100	04/1990	1990
104	Comanche Peak 2	Glen Rose	Tx.	1150	PWR	86		1992

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TABLE 0-1: NUCLEAR POWER PLANTS IN THE UNITED STATES (Continued)

							TINI	COMM
				NET	•	Z	CRIT	START
No	UTILITY/UNIT	CITY	STATE	MVe	TYPE	COMPL (1)	DATE	DATE
								•
	TOLEDO EDISON CO.							
105	* Dav1s-Besse	Oak Harbor	Ohra	874	PWR	100	08/1977	07/1978
	UNION ELECTRIC CO.							
106	* Calloway	Sulaa-	Mo.	1125	PWR	100	10/1984	12/1984
100		Fulton	mo.	1125	PWK	100	10/1704	12/1704
•	VERMONT YANKEE NUCLEAR POHER CO							
107	Vermont Yankee	Vernon	Vt.	504	BUR	100	03/1972	11/1972
	VIRGINIA POWER CO							
108	* North Anna 1	fineral	Va	915	PUR	100	04/1978	06/1978
109	* North Anna 2	Kineral	٧a. `	915	PWR	100	06/1980	12/1980
110	* Surry 1	Gravel Nack	Va.	781	PUR	100	07/1972	12/1972
311	* Surry 2	Gravel Heck	٧a.	781	PUR	100	03/1973	05/1973
	WASHINGTON PUBLIC POWER SUPPLY S	YSTEN						
112	UNP-1	Richland	Wash.	1250	PWR	65	Indefana	te
113	* WP-2	Richland	Wash.	1095	BWR	100	01/1984	12/1984
114	unp-3	Richland	Vash	1240	PUR	<b>7</b> 5	Indefini	te
	WISCONSIN ELECTRIC POWER CO.			,				
115	* Point Beach 1	Two Rivers	Urs.	485	PUR	100	11/1970	12/1970
116	* Point Beach 2	Two Rivers	Vis.	485	PWR	100	05/1972	10/1972
	WISCONSIN PUBLIC SERVICE CORP.							
117	* Kevaunee	Carlton	¥1s.	503	PUR	100	03/1974	06/1974
	WOLF CREEK MUCLEAR OPERATING COR	P						
118	* Wolf Creek	Burlington	Karis.	1135	PUR	100	05/1985	09/1985
	YANKEE ATOMIC ELECTRIC CO.							
119	* Yankee	Rove	Mass.	167	PUR	100	08/1960	07/1961

<sup>\*</sup> Units in Commercial Operation

<sup>(1)</sup> As of August 1990

TABLE D-2: NUCLEAR POWER PLANTS NO LONGER IN SERVICE

NAME	LOCATION	NET MWt	ТУРЕ	LICENSE ISSUED	SHUTDOWN
BONUS	Rincon, PR	50	BWR	4/64	6/68
CVTR	Part, SC	65	PTHW	11/62	1/67
Dresden 1	Morris, III	700	BWR	9/59	10/78
Elk River	Elk River, Mn	58	BWR	11/62	2/68
Fermt 1	Monroe, Mi	200	SCF	5/63	11/72
Fort St. Vrain	Platteville, Co	842 `	HTGR	12/73	8/89
Haliam	Hallem, Ne	25	SCGM	1/62	9/64
Humboldt Bay 3	Eureka, Ca	200	BWR	8/62	7/76 '
Indian Point 1	Buchanari, NY	615	PWR	3/62	10/74
LaCrosse	Genoa, Wi	165	BWR	7/67	4/87
Pathfinder	Sioux Fails SD	190	BWR	3/64	10/67
Peach Bottom 1	Peach Bottom, PA	115	HTGR	1/66	11/74
Piqua	Piqua, Oh	46	осм	8/62	1/66
Rancho Seco	Clay Station, Ca	913	PWR	4/75	6/89
Shippingport	Shippingport, Pa	60	PWR/ LWBR	12/57	10/82
Shoreham	Brookhaven, NY	809	BWR	<b>-</b> ·	5/89
Three Mile Island 2	Londonderry Twp, Pa	792	PWR	12/78	3/79
VBWR	Pleasanton, Ca	50	BWR	8/57	12/63

<sup>\*</sup> The Shoreham unit achieved criticality and produced power, but closed before it could begin commercial operation.

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## APPENDIX E

## QUALITY ASSURANCE CRITERIA FOR NUCLEAR POWER PLANTS AND FUEL REPROCESSING PLANTS

Quality assurance (QA) comprises all those planned and systematic actions necessary to provide confidence that a component will perform satisfactorily in service. This program must be documented by written policies, procedures, or instructions and must be carried out throughout the plant life. The QA program provides control over activities affecting the quality of components to an extent consistent with their importance to safety. The program must provide for the indoctrination and training of personnel performing activities affecting quality

The purpose of each of the 18 QA criteria is briefly explained in the following pages

<u>Criterion 1 - Organization</u> - To identify all activities affecting quality and to assure that the responsibilities and authorities of key personnel are clear

<u>Criterion 2 - Quality Assurance Program</u> - To cause the project manager to articulate the actions necessary to plan and implement an effective quality assurance program

<u>Criterion 3 - Design Control</u> - To control the following processes in accordance with the requirements of Applicable and Relevant or Appropriate Requirements: (1) designing tests and sampling patterns to characterize the geologic setting, to develop models to predict the performance and long-term stability of the site, and to predict the environmental interaction between the site and its surroundings; (2) specifying requirements for design and construction, and (3) designing computer codes.

<u>Criterion 4 - Procurement Document Control</u> - To provide the management controls to manage the work activities of contractors and subcontractors and ensure acceptable quality of the results

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<u>Criterion 5 - Instructions, Procedures, and Drawings</u> - To ensure the use of formal instructions for work activities related to the accomplishment of performance objectives and the design bases.

<u>Criterion 6 - Document Control</u> - To ensure that documents prescribing activities related to the accomplishment of the performance objectives and the design bases are controlled during review, approval, and distribution to ensure that those performing activities use approved and up-to-date instructions.

<u>Criterion 7 - Control of Purchased Material, Equipment, and Services</u> - To oversee and control the work of contractors and suppliers and to ensure that the results are consistent with performance objectives and design bases.

<u>Criterion 8 - Identification and Control of Materials, Parts, and Components</u> - To ensure that all materials, parts, samples, and components important to the accomplishment of performance objectives and the design bases are identified and controlled

<u>Criterion 9 - Control of Special Processes</u> - To ensure that all work activities important to the accomplishment of performance objectives and the design bases are controlled, including the identification of activities that require specially trained personnel, or specialized equipment or procedures.

<u>Criterion 10 - Inspection</u> - To ensure that independent, pre-planned inspections are performed where it is deemed necessary to establish the acceptability of a product, process, or service, either in progress or upon completion

<u>Criterion 11 - Test Control</u> - To ensure that test are conducted to determine if an item or service is acceptable or to satisfy a need for more information

<u>Criterion 12 - Control of Measuring and Test Equipment</u> - To ensure that measurements that affect quality of work are taken only with instruments, tools, gauges, or other measuring devices that are accurate, controlled, calibrated, and adjusted at predetermined intervals to maintain accuracy within necessary limits.

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<u>Criterion 13 - Handling, Storage, and Shipping - To ensure control over handling, storage, cleaning, packaging, preservation, and shipping of items affecting quality of work.</u>

<u>Criterion 14 - Inspection, Test, and Operating Status</u> - To ensure the identification of the inspection and/or test status of samples, structures, systems, and components to prevent inadvertent use of items found to be unacceptable for use.

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<u>Criterion 15 - Nonconforming Materials, Parts, or Components</u> - To ensure that items not conforming to specified requirements are identified and controlled to prevent inadvertent use

<u>Criterion 16 - Corrective Action</u> - To ensure that management systems comprised by the QA program are constantly monitored and that timely measures are taken to correct conditions adverse to quality.

<u>Criterion 17 - Quality Assurance Records</u> - To ensure that records important to the accomplishment of performance objectives and the design bases (including the data analysis phase, hearings, permitting and licensing processes) are sufficient to demonstrate the quality of work performed Records will also be needed should problems related to the performance of the facility occur at a later date.

<u>Criterion 18 - Audits</u> - To ensure that audits, which are part of the management system's sensors, are effective by being well planned, conducted by trained personnel familiar with the work being audited, and designed to measure the potential of the activity or process being audited to produce an acceptable product

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